

FPPC , March 4, JAERI-DOE

JT-60U Status and Plans

M. Kikuchi for the JT-60 Team

Japan Atomic Energy Research Institute

1. Unification of JAERI and JNC

JAERI (Japan Atomic Energy Research Institute) and JNC (Japan Nuclear Fuel Cycle-Development Institute) will be unified to form a new entity in Mid 2005. Present staff and budget of two institutes as of 2003 are as follows.

**JAERI employs ~2,200 staff
budget ~ 94 billion Yen.**

**JNC employs ~2,300 staff
budget ~ 130 billion Yen.**



**President of JAERI
T. Okazaki from
JAERI HP**



Procedures for personal assignment

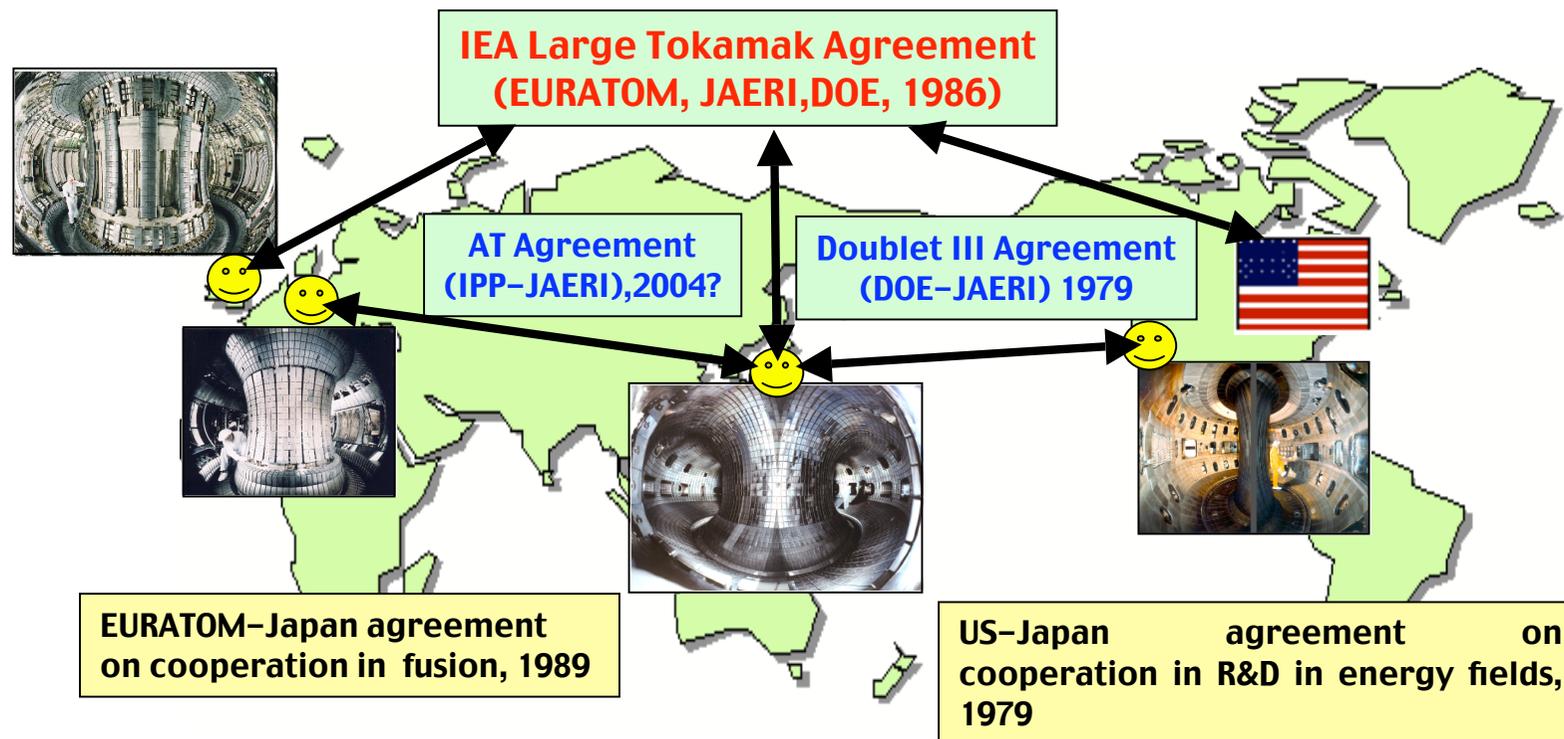
ITPA Joint experiments

US to JT-60U : LT IA Appendix A2 signed by E. Oktay and Visitor

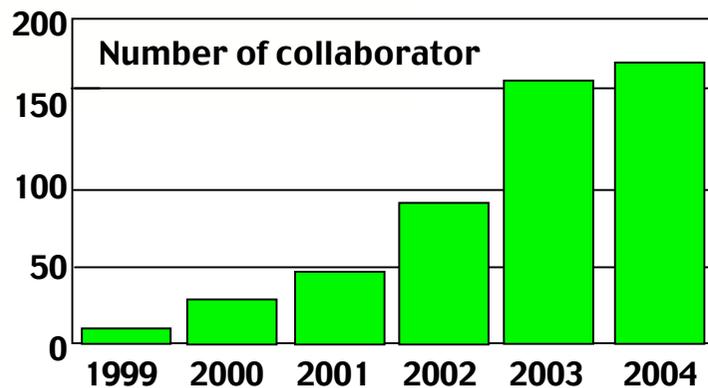
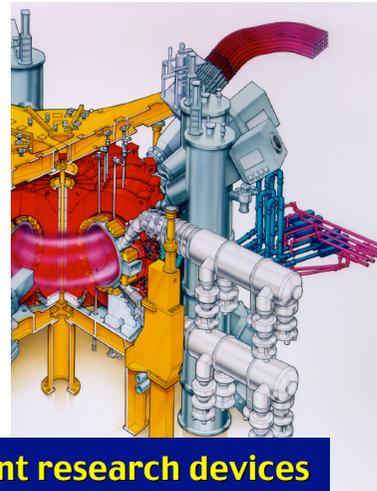
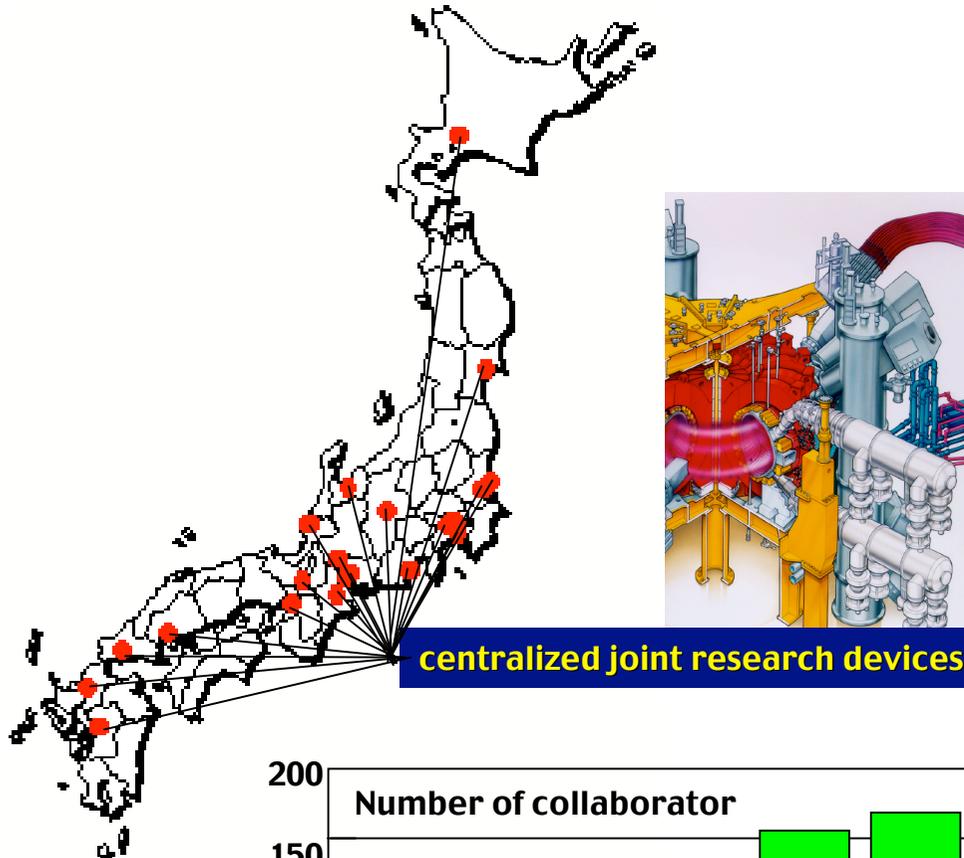
EU to JT-60U : LT IA Appendix A2 signed by M. Watkins and Visitor

JT-60U to US : Personal assignment agreement (US-J) is required except PPPL and GA.

JT-60U to EU : Personal assignment agreement (EU-J) is required except JET.



National research collaboration



Research Theme structure for 2003–2004

Director of Department of Fusion Plasma Res.

Theme Leaders; (T. Fujita), S. Ide, H.

Task

Extension of Discharge Duration (S. Ide)

Development of advanced tokamak(N. Sakamoto)

Confinement improvement at high density (H. Takenaga)

AMTEX (K. Tsuzuki)

Subject

Core Plasma (T. Takizuka)

H-mode Physics/ Pedestal (Prof. K. Toi(NIFS))

Modelling of ITB (Prof. A. Fukuyama(Kyoto U))

Current Hole (T. Fujita)

MHD/Energetic Particle / Disruption, (M. Takechi)

Current Drive (Prof. K. Hanada(Kyushu U))

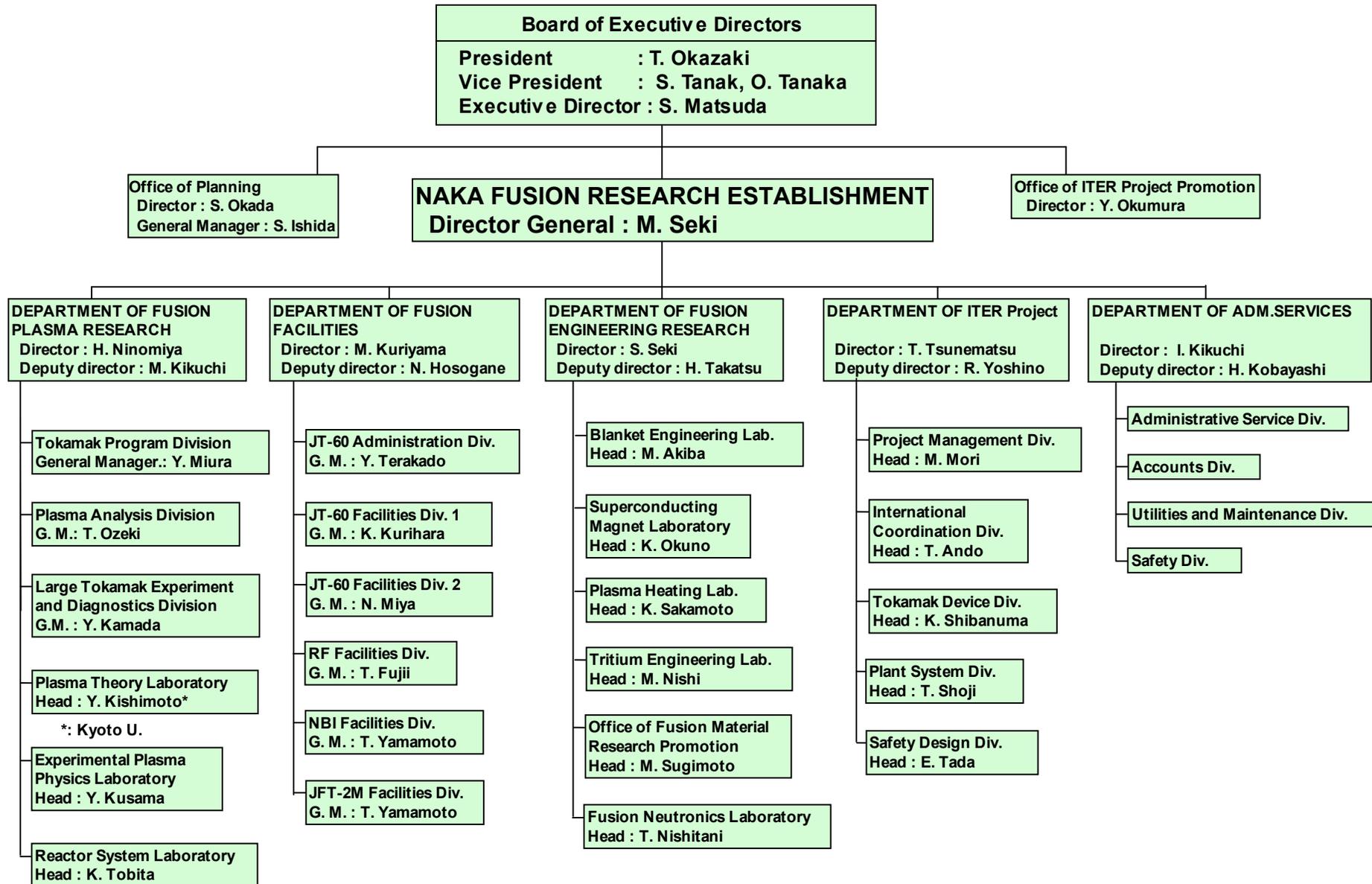
Divertor/SOL (N.Asakura)

Advanced Fueling (H. Ogawa)

Innovative operation (Prof. Y. Takase(U. of Tokyo))

Plasma Material Interaction (Prof. T. Tanabe(Nagoya U))

3. Organization of Fusion Research in JAERI



Introduction

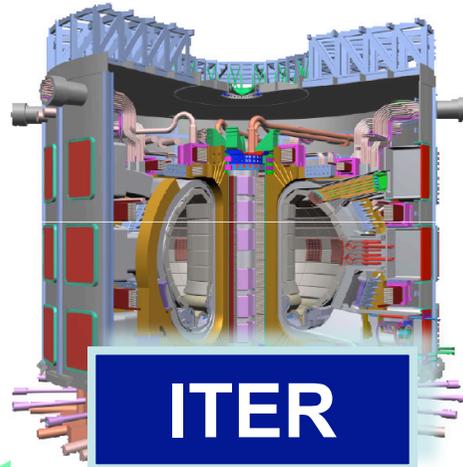
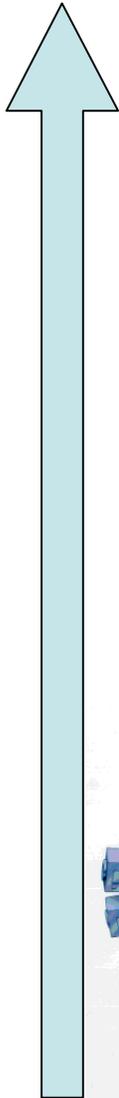
JT-60U

●ITER Physics R&D

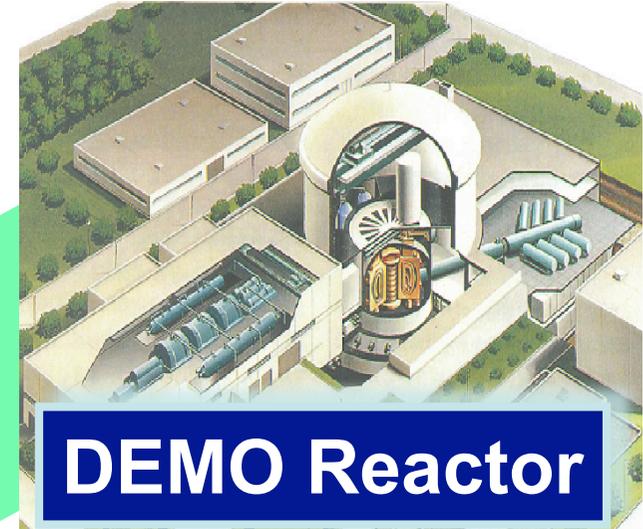
Objectives

●Advanced Tokamak Concepts for ITER & DEMO

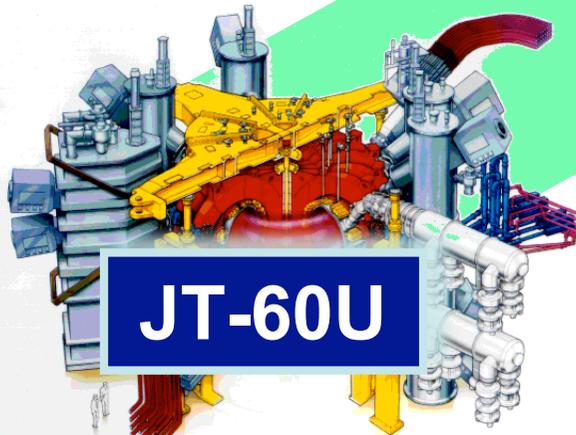
Performance



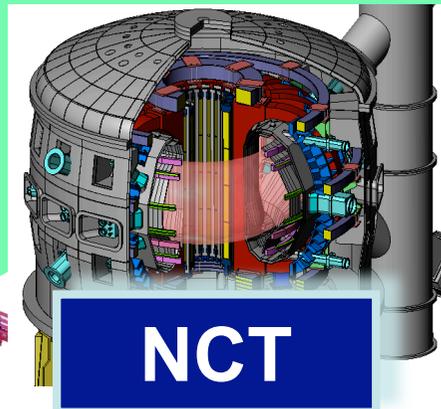
ITER



DEMO Reactor



JT-60U



NCT

4. Status of JT-60U

Recent trends of operation cycles

FY2000 : 9 cycles two shifts 18 weeks of experiment

FY2001 : 6 cycles two shifts 12 weeks of experiment

FY2002 : 2 cycles two shifts 04 weeks of experiment

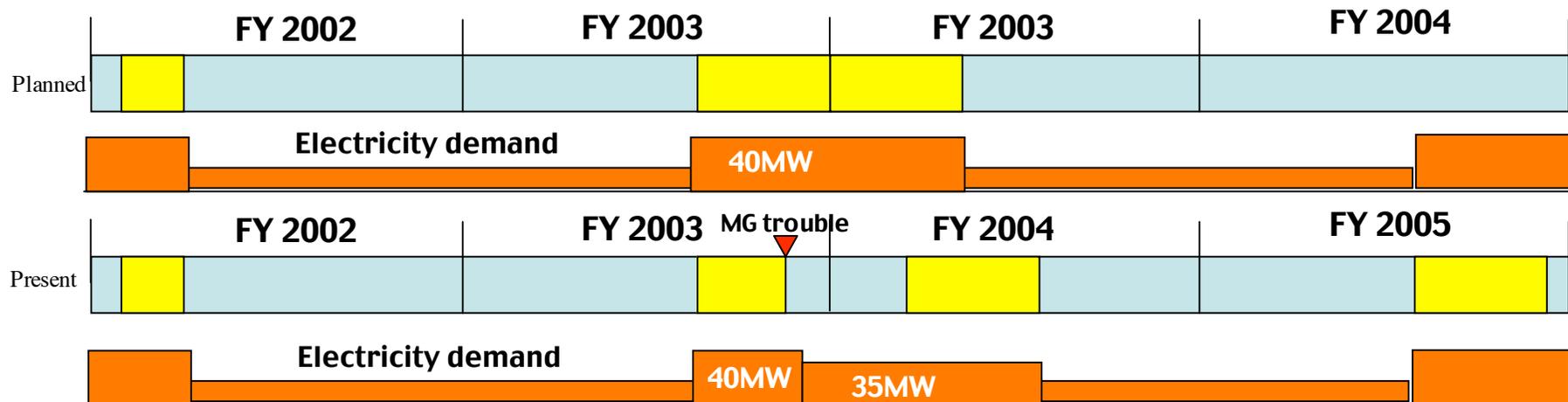
FY2003 : 2 cycles one shift 08 weeks of experiment

MG trouble in Feb. 2004 3 EX weeks cancelled.

FY2004 : 2 cycles one shift 08 weeks of experiment 65s
with reduced Bt capability

FY2005 : 2 cycles one shift 08 weeks of experiment 65s

Research plans are based on two year frame.

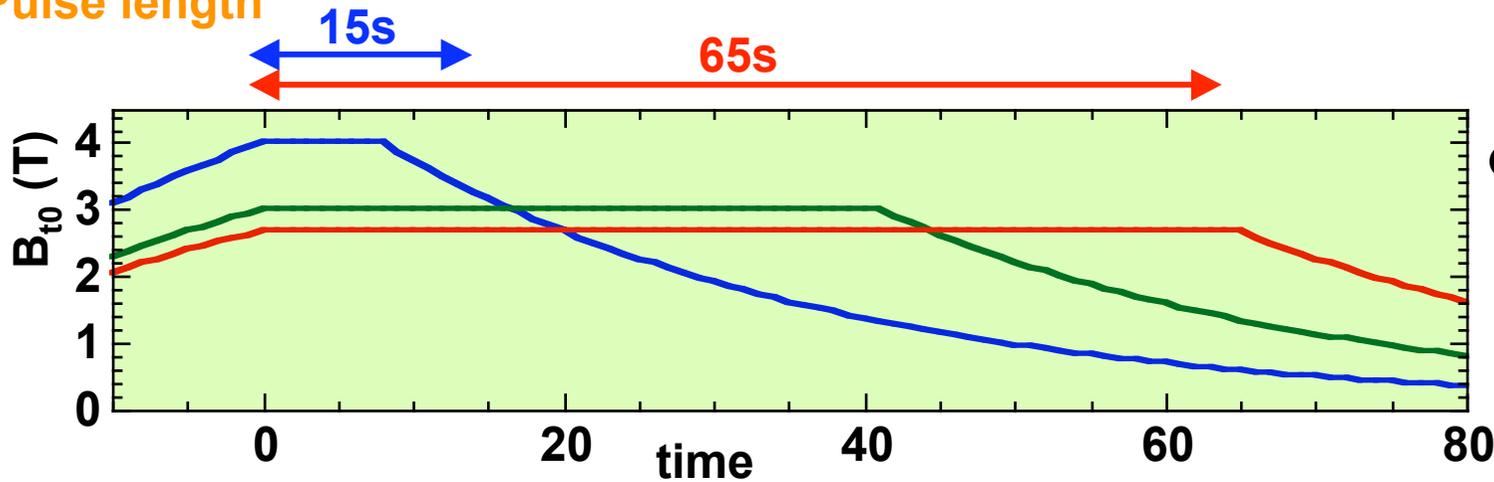


1. Modification for long pulse discharge

Max. pulse length of a discharge : 15s \Rightarrow **65s**.

Modification on control systems in operation, Heating/CD and diagnostics, but not on major hardware.

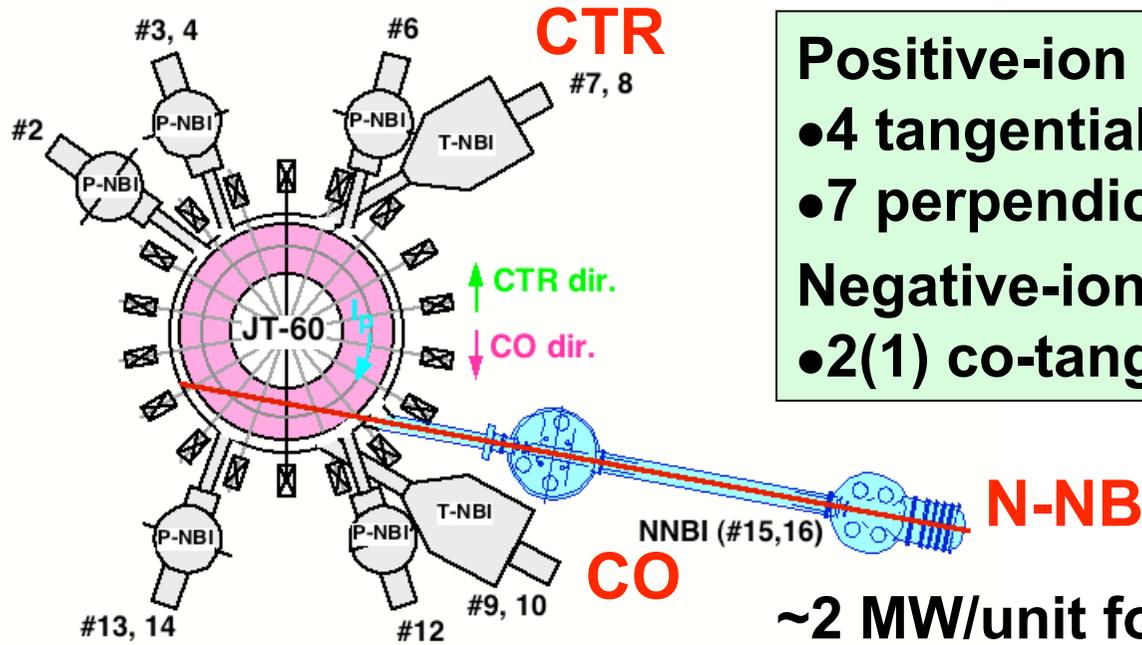
Pulse length



● B_{t0} flat top
4T \times 8s \Rightarrow
 \sim 2.7T \times 65s

- Control of NB: 30s, control of RF: 60s.
- PFC temperature is not actively cooled; carbon tiles (CFC for divertor), a small number of W tiles, and RF antennas.
- Vessel temperature is kept 150-300 degC; mostly 150 degC in 2003-04 campaign.

Improvement in NB systems (P-NB, N-NB)



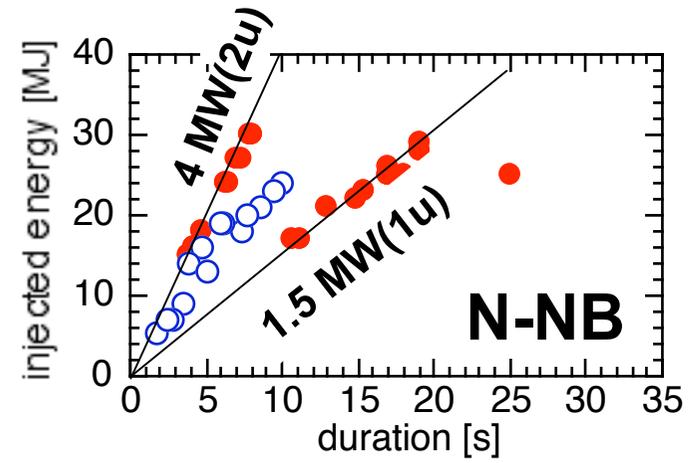
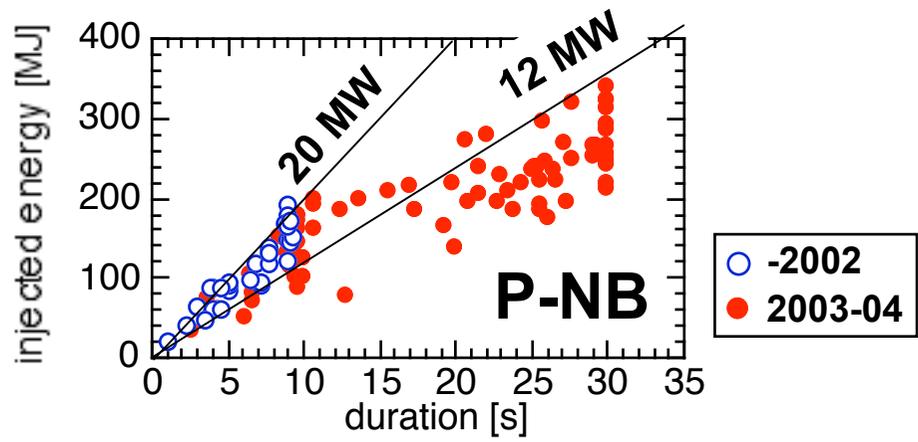
Positive-ion NB (P-NB)(~85keV)

- 4 tangential (2 co + 2 ctr): 30 s
- 7 perpendicular: 10s(6u),15 s(1u)

Negative-ion NB (N-NB)(~360keV)

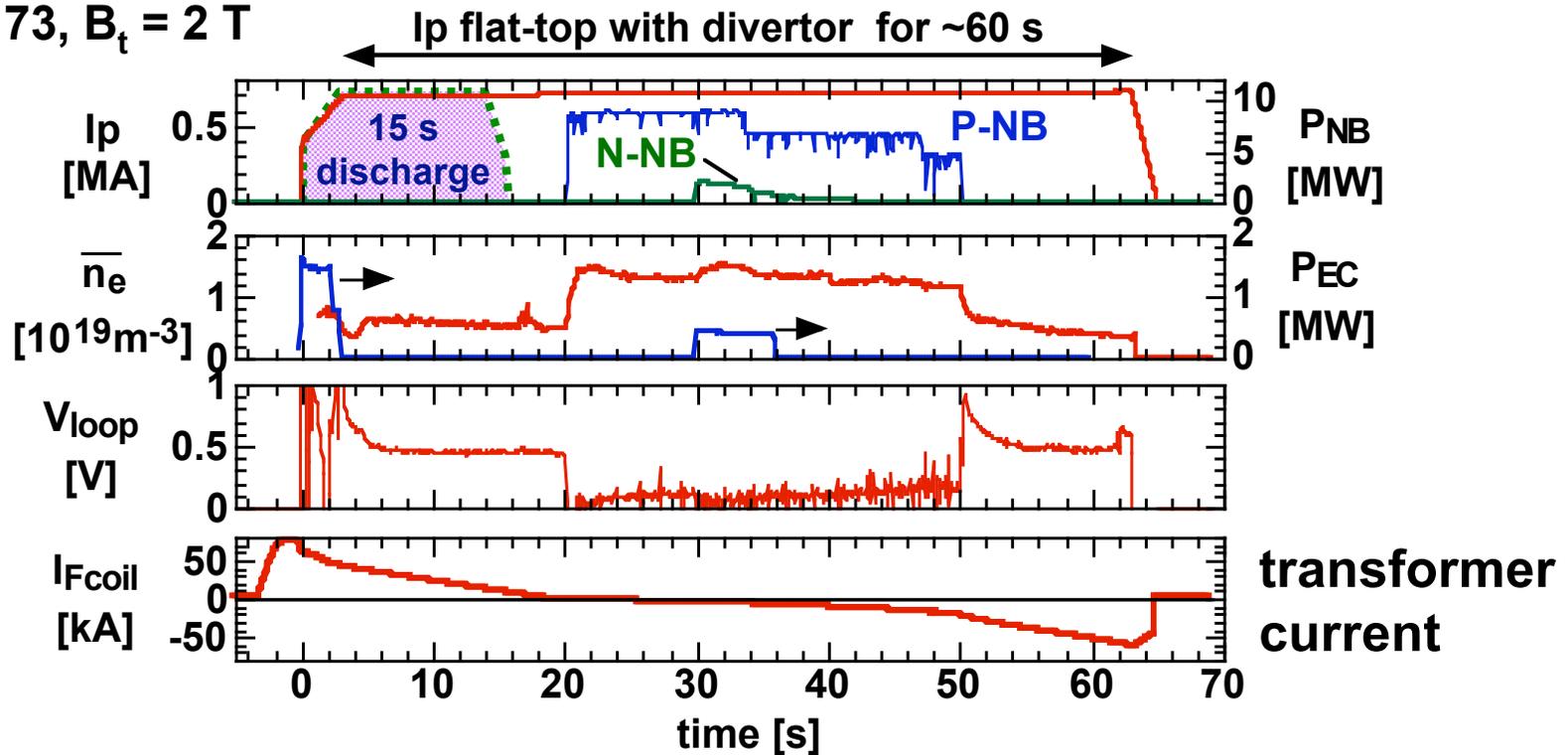
- 2(1) co-tangential: 10 s (30s)

**~2 MW/unit for long pulse operation.
350 MJ achieved with P-NB + N-NB**



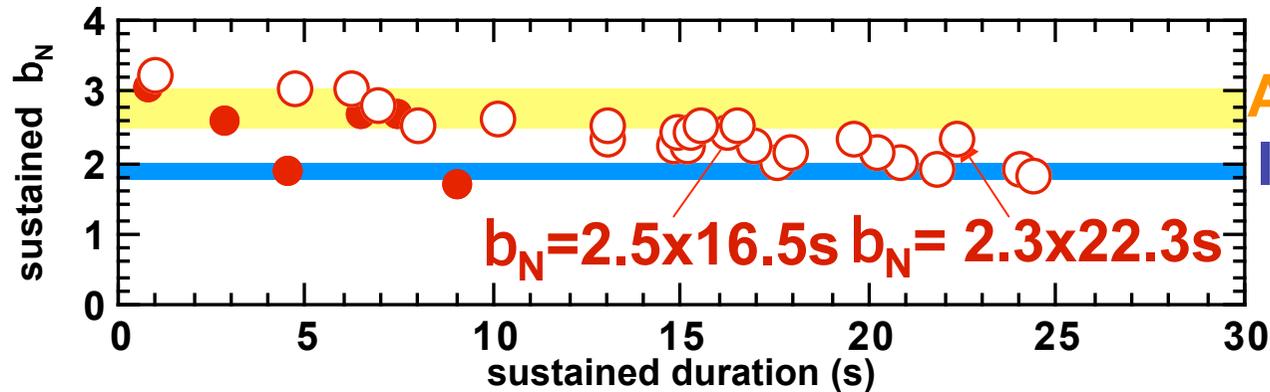
65 s discharge has been achieved

E43173, $B_t = 2$ T

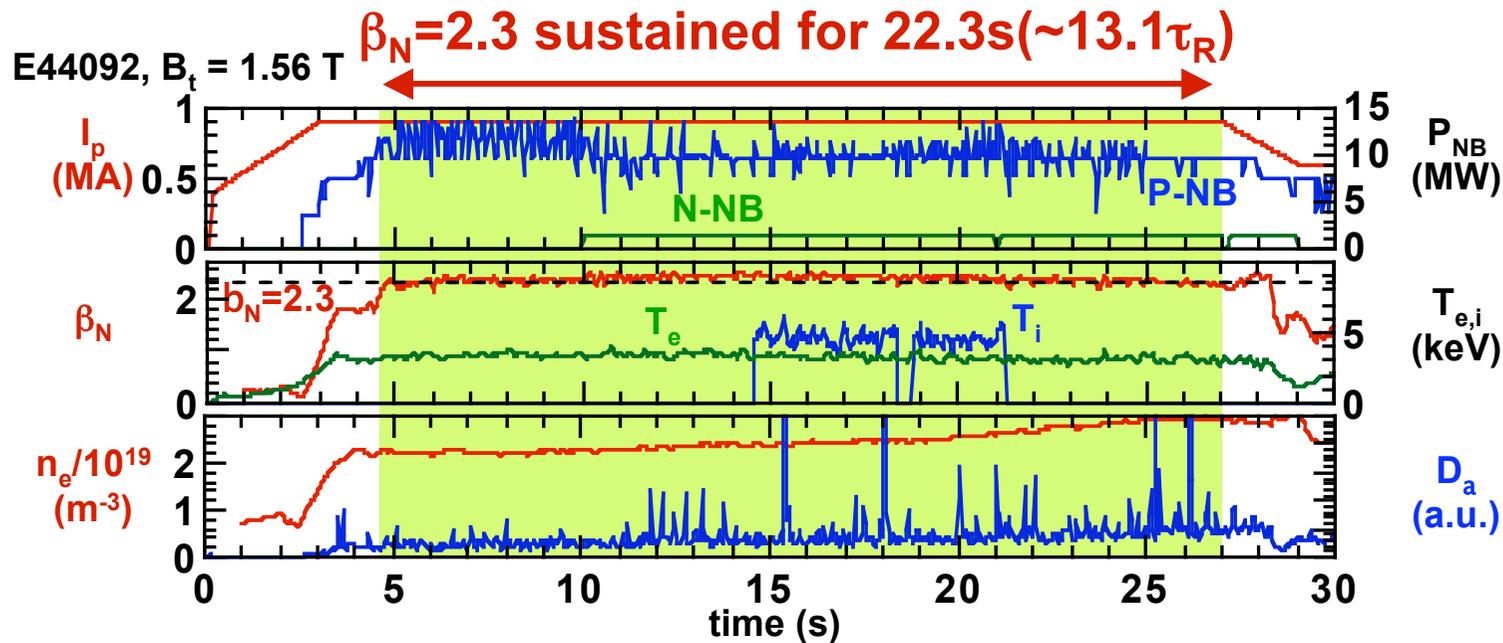


- Flux swing of transformer saved by NB+EC heating and EC breakdown assist.
- Long pulse LHCD will result in higher I_p .
- For 30 s, 1.4 MA ELMy H-mode was maintained.

Long sustainment of high β_N in extended pulse operation



ITER
Advanced Op.
Inductive Op.



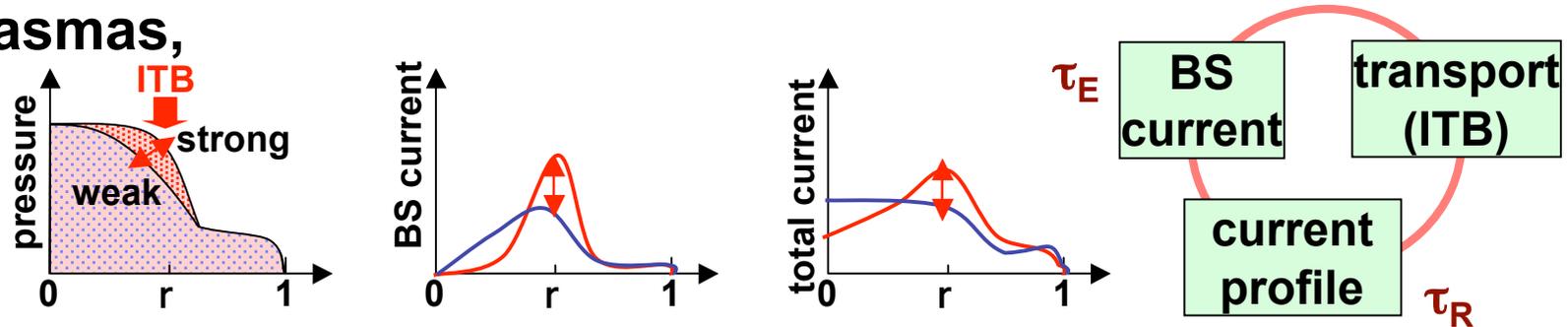
$q_{95} \sim 3.1-3.2$

$H_{89PL} \sim 1.9$

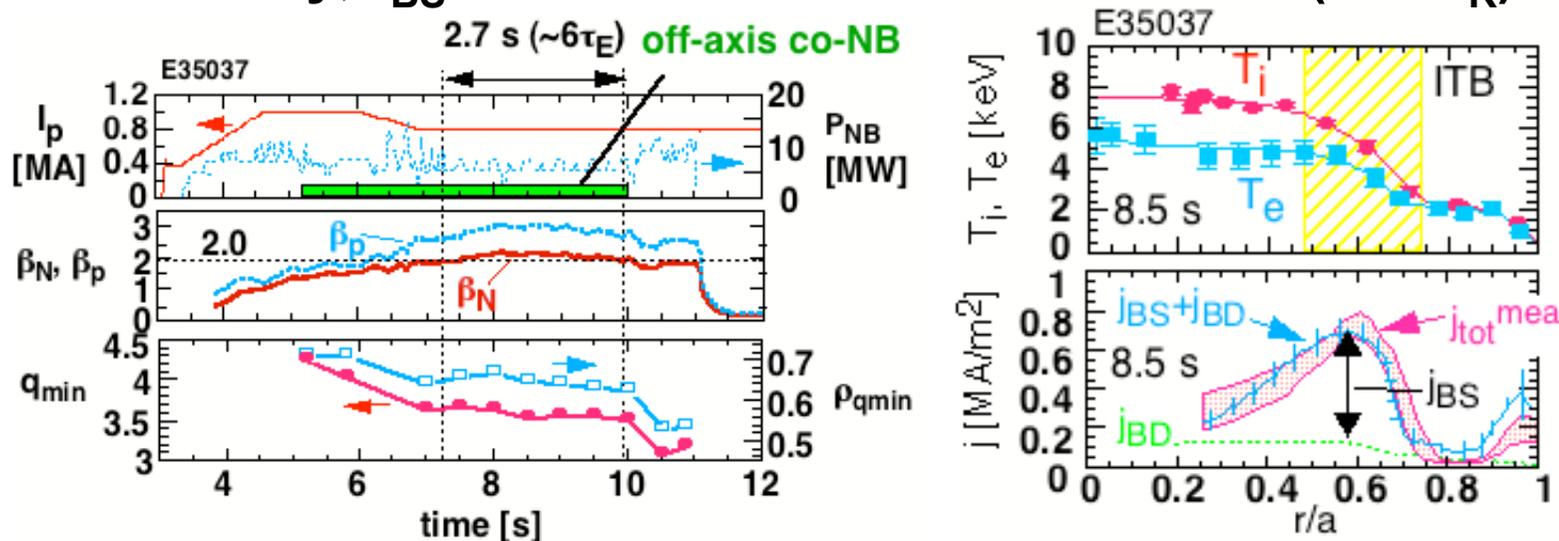
$\tau_R \equiv m_0 \langle \sigma_{NC} \rangle a^2 / 12$ D.R. Mikkelsen, Phys. Fluids B 1 (1989) 333.

Long sustainment of high f_{BS}

- Strong linkage of $j_{BS}(r)$, $j_{total}(r)$ and transport in high f_{BS} plasmas,



- Previously, $f_{BS} \sim 80\%$ was maintained for 2.7 s ($\sim 0.6\tau_R$)

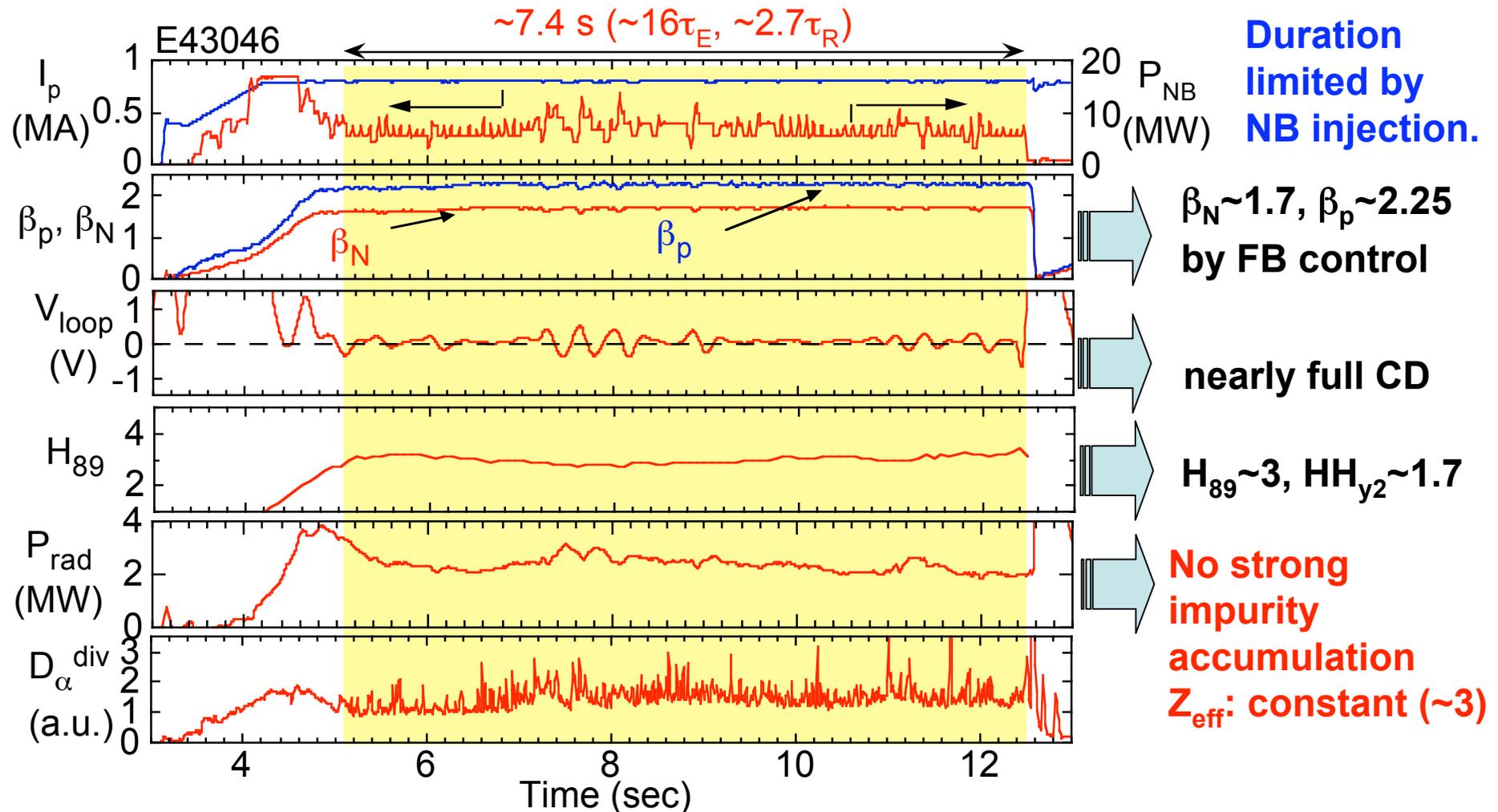


- Control must be demonstrated in longer sustainment ($\gg \tau_R$).
When ITB (c) changes \Rightarrow p and j_{BS} changes in τ_E . But j_{tot} changes in τ_R .

$\beta_p \sim 2.25$, $HH_{y2} \sim 1.7$ sustained for $\sim 7.4s$ ($\sim 2.7\tau_R$) under nearly full CD in RS plasma

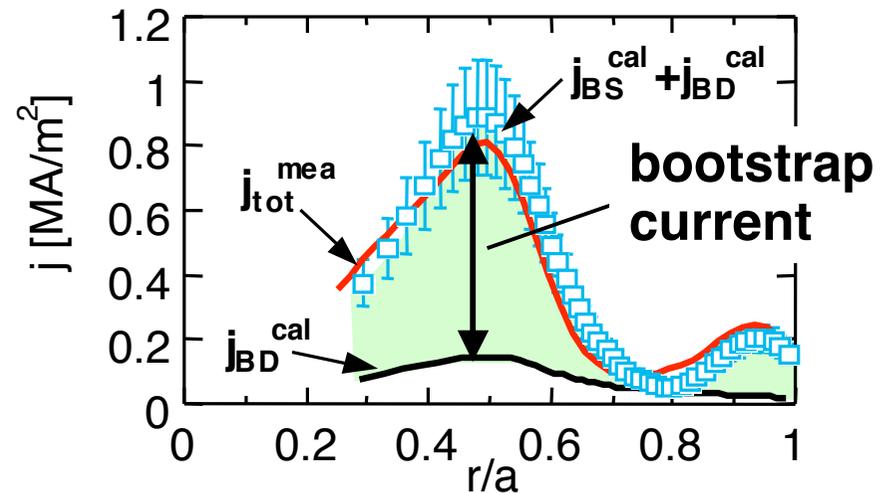
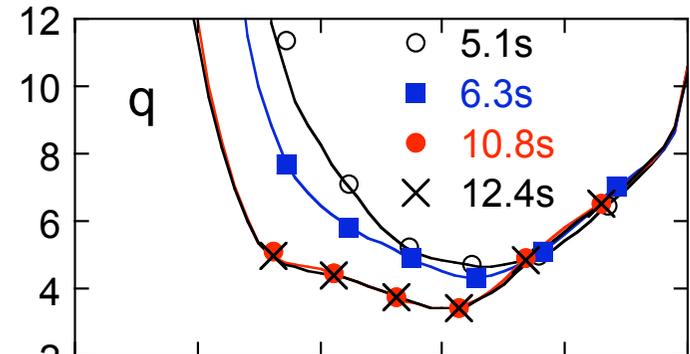
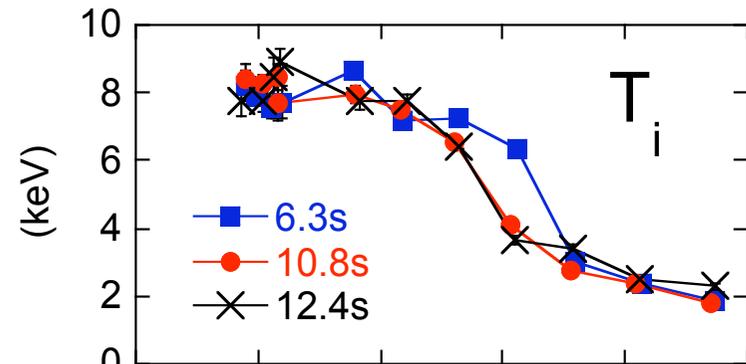
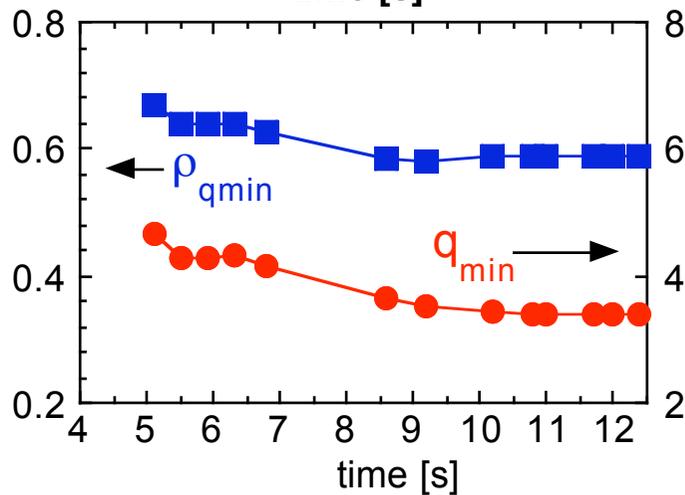
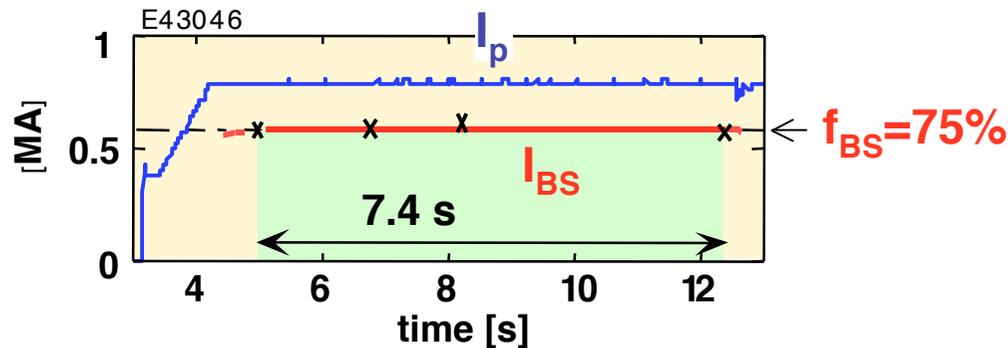
Reversed shear ELMy H-mode (3.4T, 0.8MA, $q_{95} \sim 8.6$, $\delta \sim 0.42$)

Non-inductive CD: Bootstrap dominant & $P_{NB}^{inj}(co) = 3.2MW$



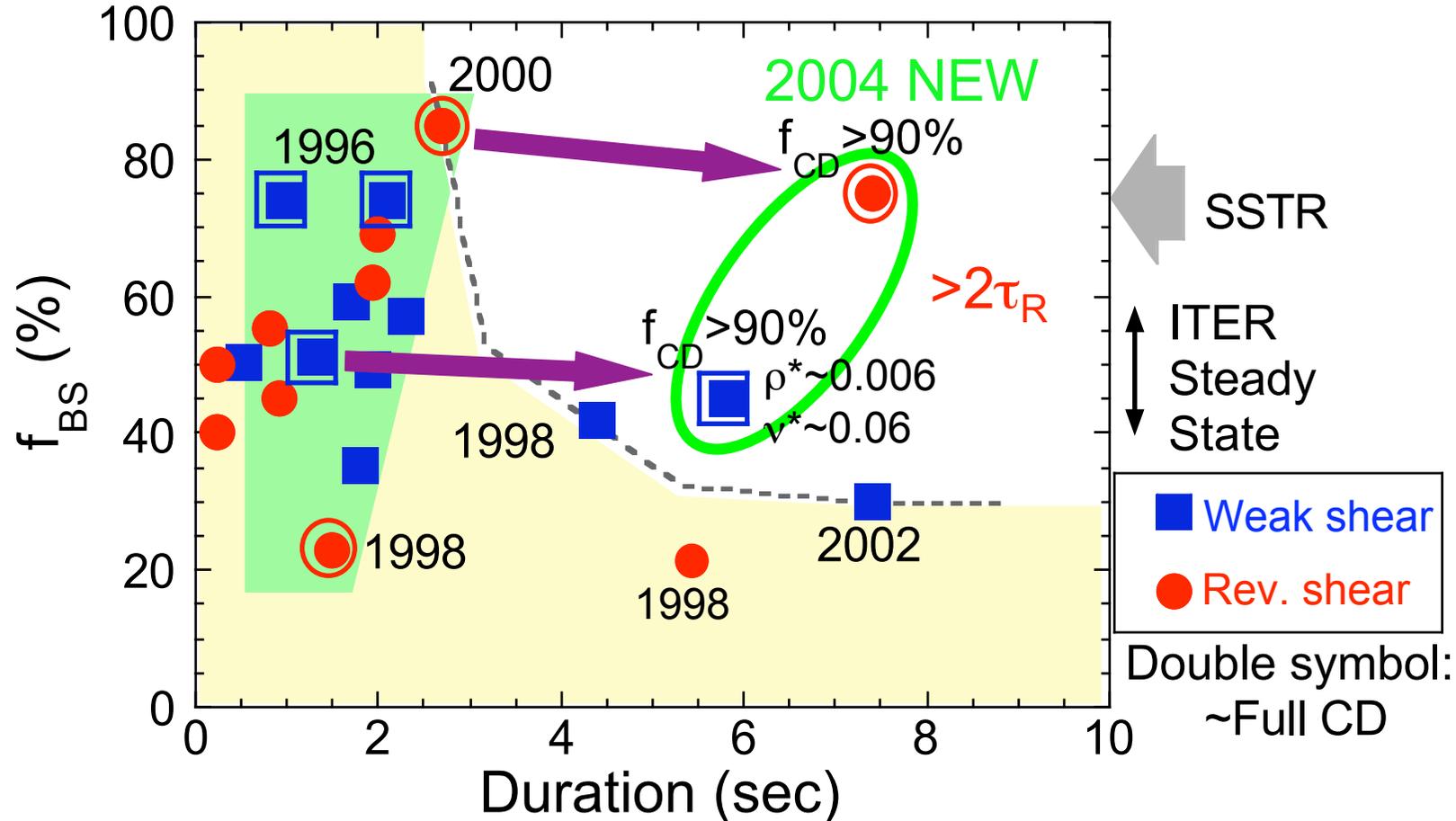
$f_{BS} \sim 75\%$ sustained for $\sim 7.4s$ ($\sim 2.7\tau_R$)

- BS current peaked at the ITB.
- $f_{BS} \sim 75\%$, $f_{BD} \sim 20\%$, $f_{CD} \sim 95\%$
- well-aligned BS current
- $j(r)$ and $p(r)$ reached stationary conditions.

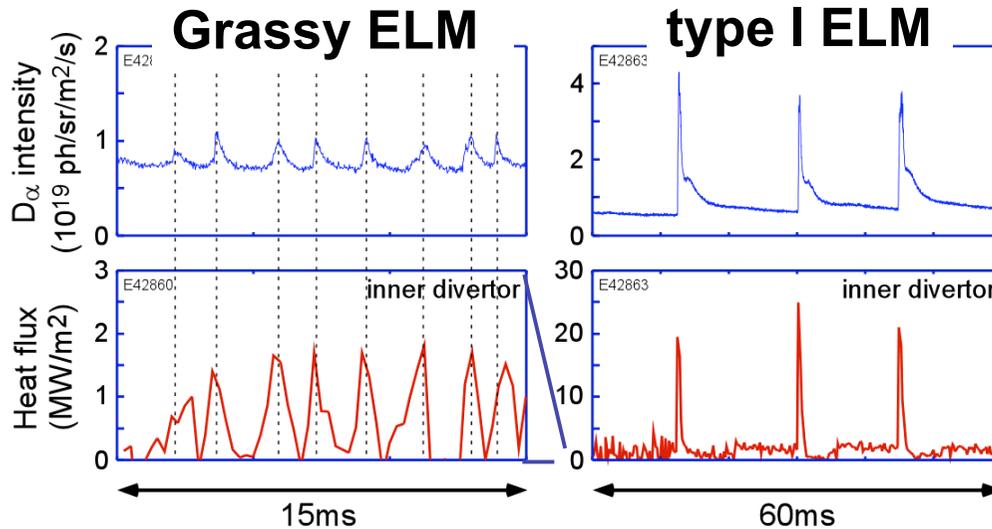


Long sustainment of nearly full CD plasmas

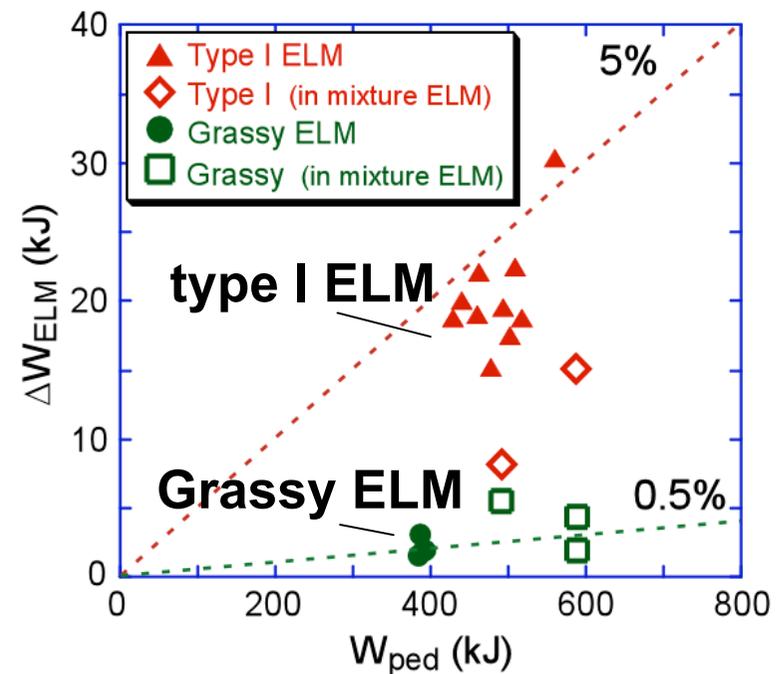
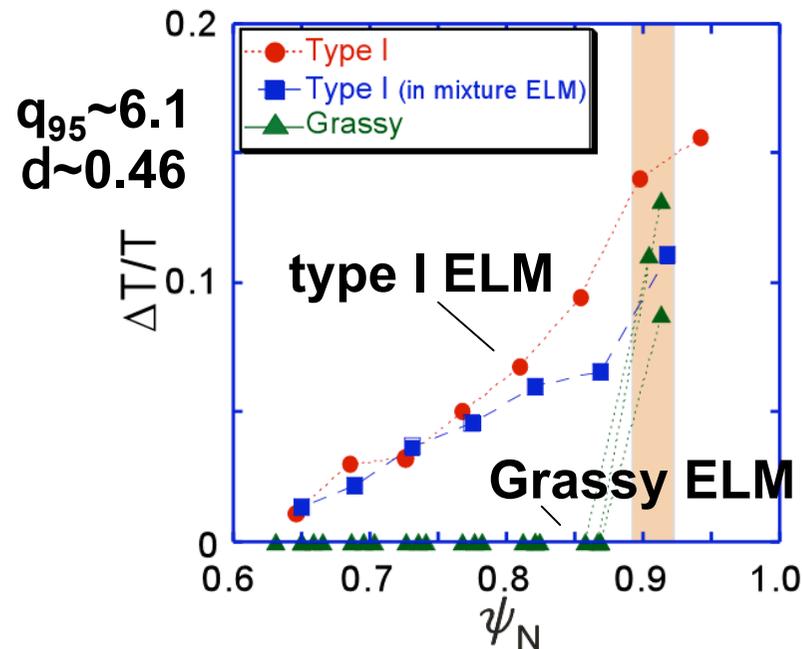
- Achieved region of \sim full CD with large f_{BS} has been significantly extended.
- $j(r)$ approached stationary conditions.
- Controllability of $j(r)$ and $p(r)$ must be studied in a long time scale.



Divertor heat load is reduced by a factor of 10 during grassy ELM than during type I ELM



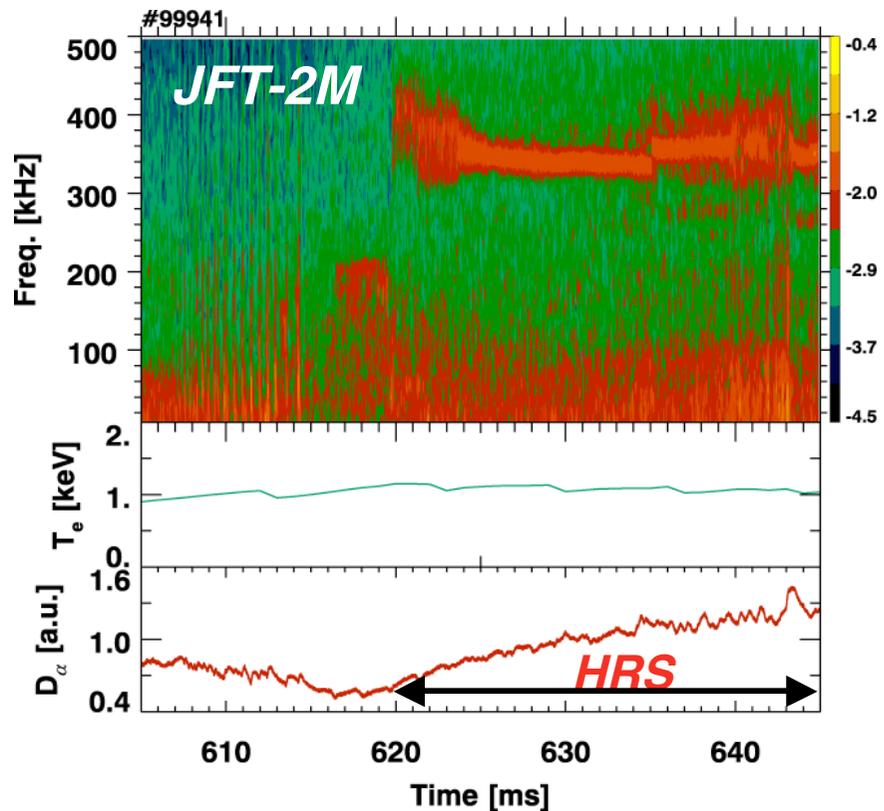
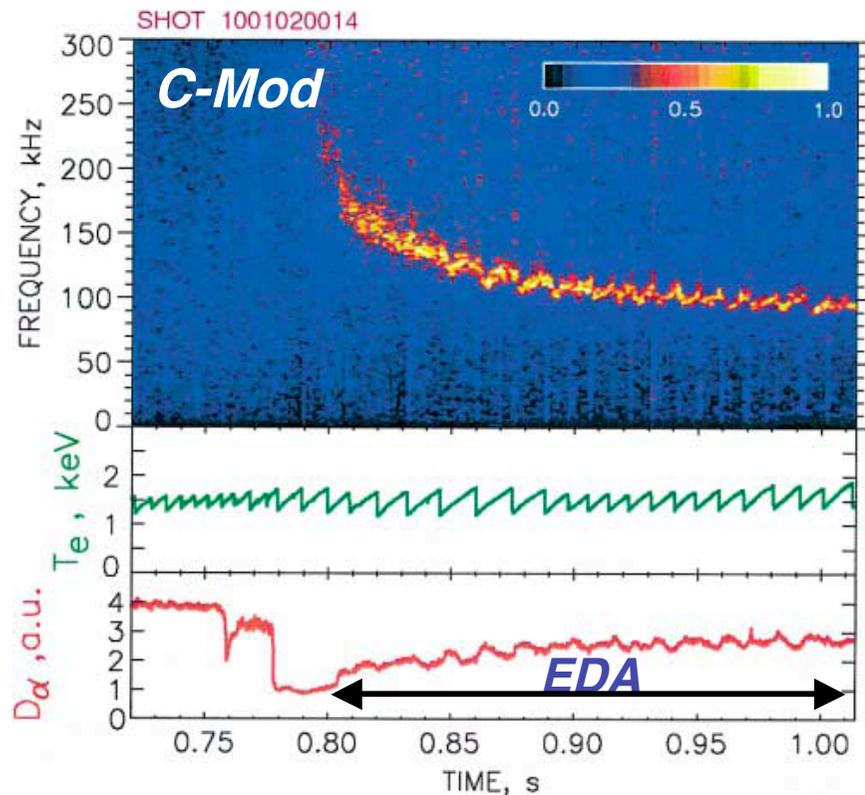
- ΔW_{ELM} is reduced by a factor of 10 during grassy ELM.
- Narrower radial extent.
- Low n_e^* region ($n_e^* \leq 0.15$).



Inter-machine comparison between C-Mod EDA and JFT-2M HRS regimes

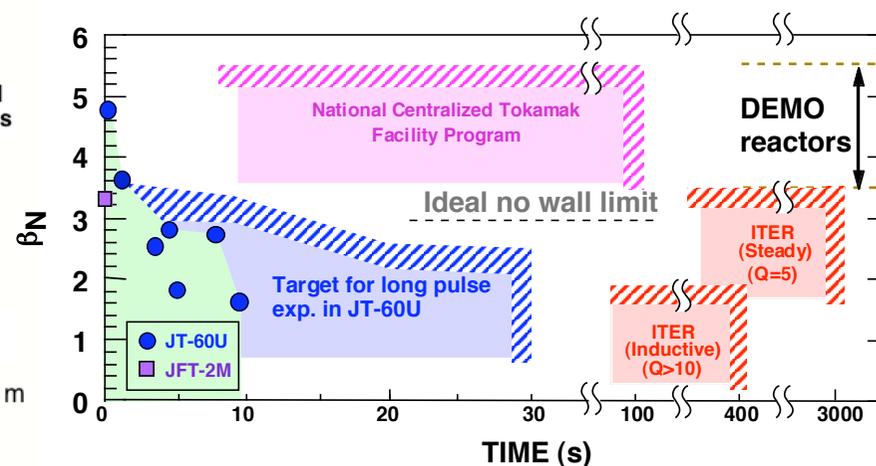
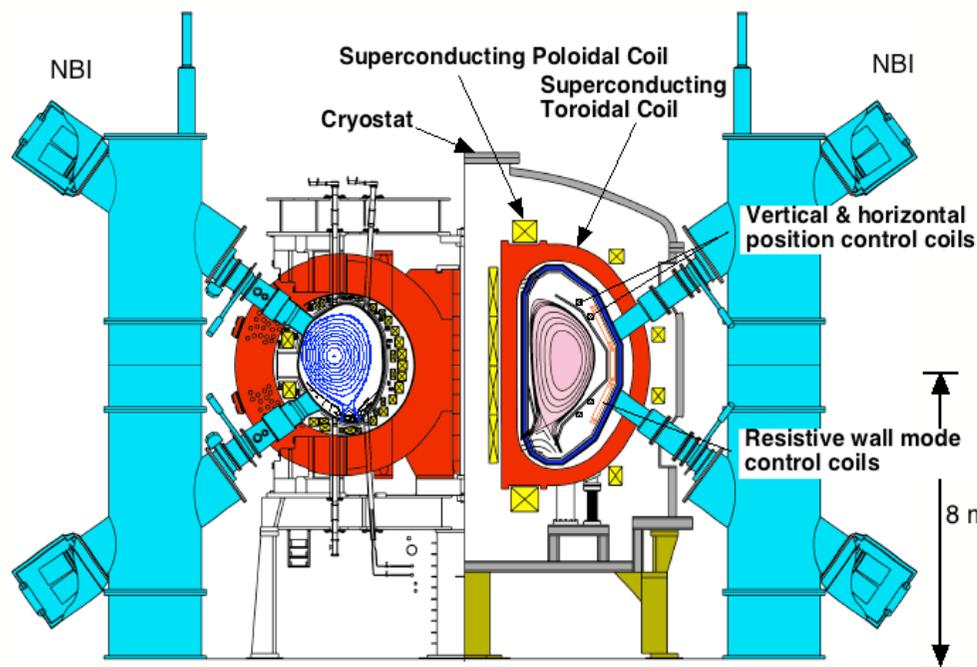
JFT-2M has found a “**H**igh **R**ecycling **S**teady” (**HRS**) H-mode
 These features are qualitatively similar to **C-Mod EDA** regime
 ITPA Inter-machine collaboration was proposed (*PEP-12*).

Questions: *Is this the same regime as EDA? If so, how do access conditions and fluctuations scale and compare?*



National Centralized Tokamak Program

- **Objectives:** to realize high-beta steady-state operation with the use of reduced radio-activation ferritic steel in a collision-less regime.
- **Planning :** For further progress in the high beta steady state research, the modification of JT-60 is regarded as “National centralized tokamak facility program”. Detailed design work is ongoing in collaboration with universities, institutes and industries in Japan.



Summary

- **Long pulse operation is successfully performed.**
 - 65 second discharge
 - 30 second H-mode
 - 24 second sustainment of $\beta_N \sim 1.9$
- **Region of sustained high beta has been extended.**
 - Stationary RS ELMy H-mode plasma with $f_{BS} \sim 75\%$ has been sustained for 7.4s under near full CD
- **NCT is being promoted.**
 - DOE-JAERI technical planning of tokamak experiments(WS)
- **US-Japan cooperation is always productive.**
 - C-MOD/JT-60U joint experiments