

The U.S. Collaboration on the Wendelstein 7-X Stellarator

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Outline

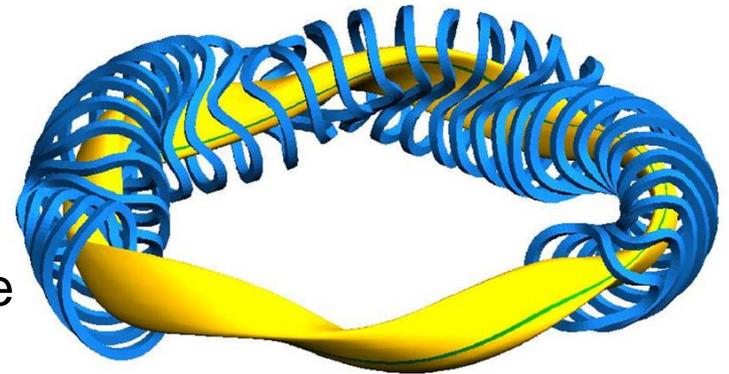
- What is W7-X?
- U.S. Research Agenda
- Program opportunities and needs going forward

What is Wendelstein (W7-X)?

- A new 3D toroidal magnetic confinement experiment (a “stellarator”)
- Large-scale test of an innovative computer-optimized machine design.
 - $R_{\text{maj}} = 5.5 \text{ m}$, $\langle a \rangle = 0.5 \text{ m}$, $B_0 = 3 \text{ T}$, $P_{\text{heat}} \leq 20 \text{ MW}$
- Capability for long-pulse ($\leq 30 \text{ min.}$) plasma operation.

New physics

- Can it sustain a stable, high performance plasma for long times as predicted?
- Can plasma boundary conditions be made compatible with both high plasma performance and long wall material lifetimes?



A key element of EU’s roadmap to fusion electricity:

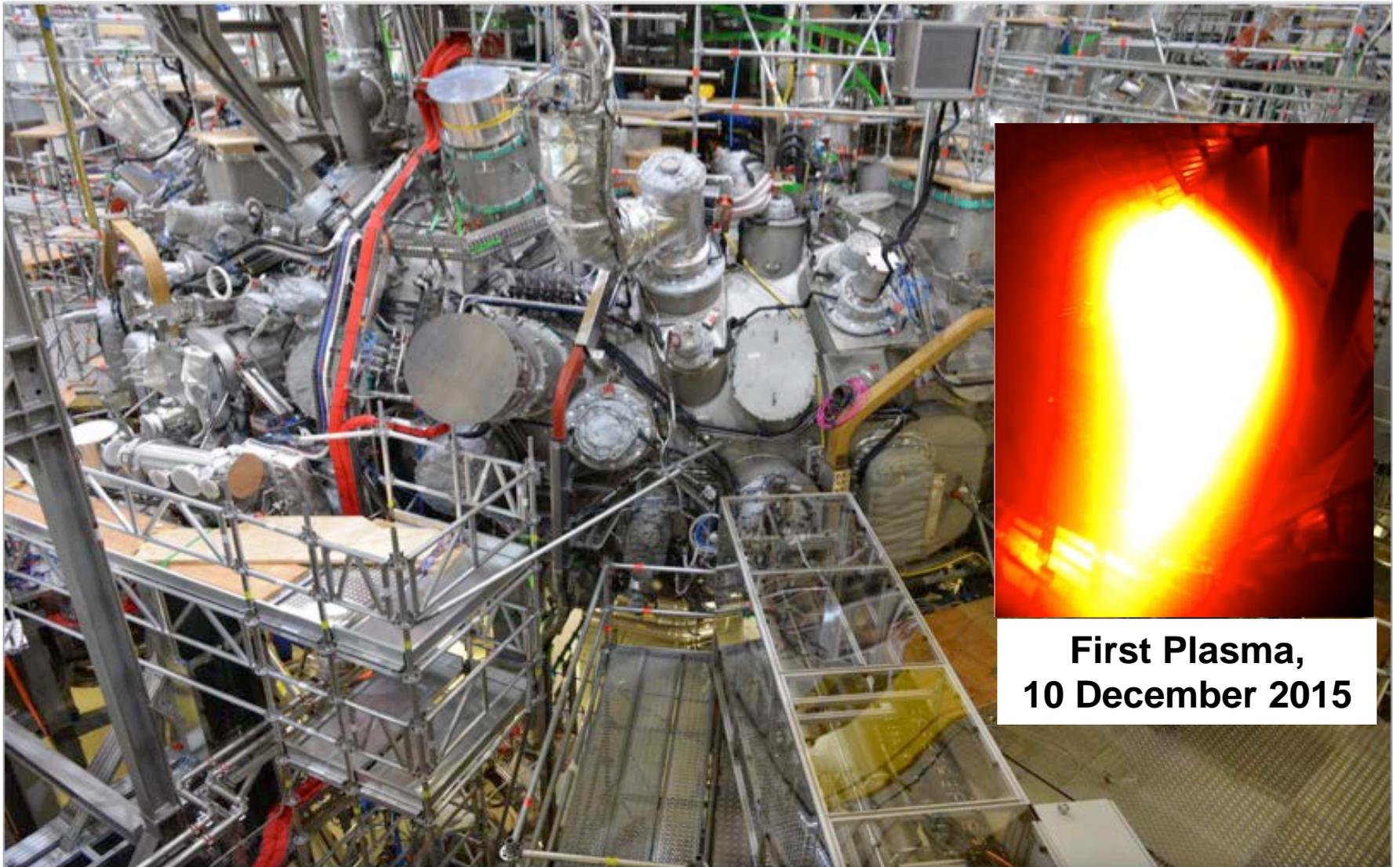
- Mission 8: “scientific exploitation of the W7-X experiment in validating the energy and particle confinement of optimised stellarators and qualifying the island divertor.”

U.S. Interests in Wendelstein 7-X

- Opportunity to deepen understanding of 3D plasma physics using the world's most advanced stellarator.
- Opportunity to advance long-pulse PMI science in a toroidal confinement system.
- Opportunity to advance an ITER-relevant model for research collaboration on a multi-national, Europe-sited facility.

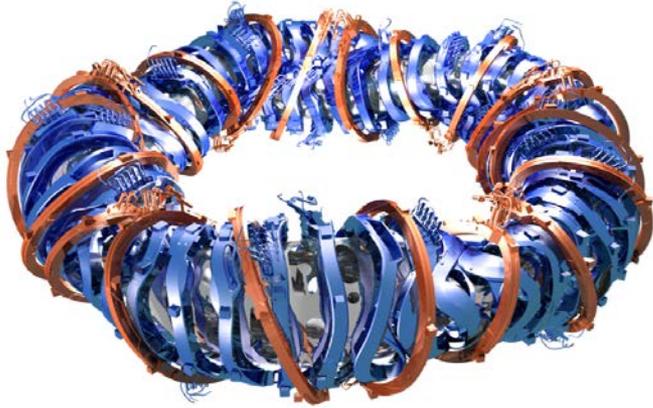
Advances FES goals under Long Pulse Burning Plasma Science.

The W7-X Machine is built ...and operating!



**First Plasma,
10 December 2015**

Vacuum Flux Surface Measurements Confirm the Accuracy of the 3D Superconducting Magnet System



- 50 non-planar coils
- 20 planar coils
- 7 independent circuits.

Tested to 2.5 T.

- Intrinsic island chain geometry (size, phase, helicity) matches predictions.
- No significant error field detected.



The U.S. Research Agenda for W7-X

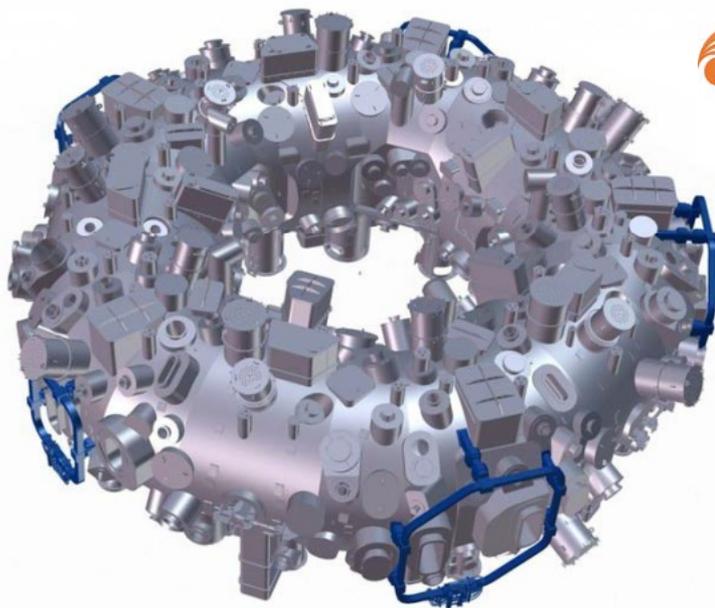
Theme: Control of high-performance steady-state 3D plasmas.

- How well can we diagnose and control the 3D equilibrium from start-up thru high- β steady-state?
- Can we control the divertor well enough to:
 - keep the heat exhaust flowing uniformly to high heat-flux surfaces while protecting sensitive components?
 - control impurities and maintain plasma purity?
- What governs transport?
- Can we control the density profile for optimum performance?

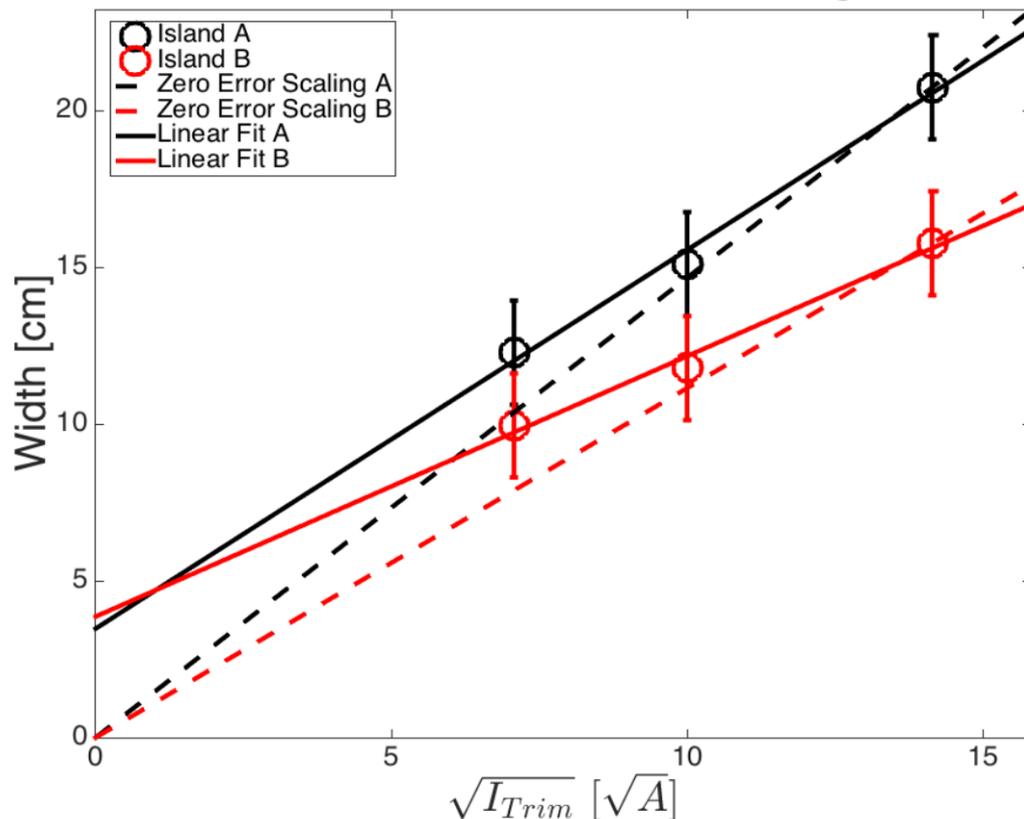
Carrying out the U.S. Agenda...

Diagnosis and control of 3D equilibrium

U.S. supplied the W7-X trim coil system *and* led experiments showing that they affect the magnetic configuration as expected



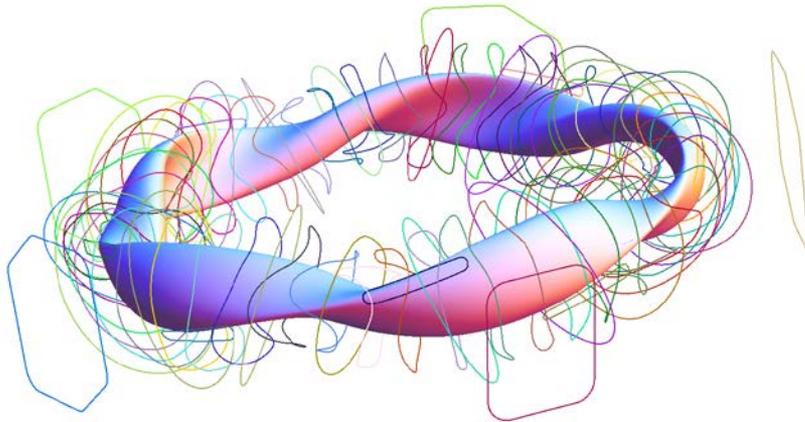
iota=1/2 Island Width Scaling



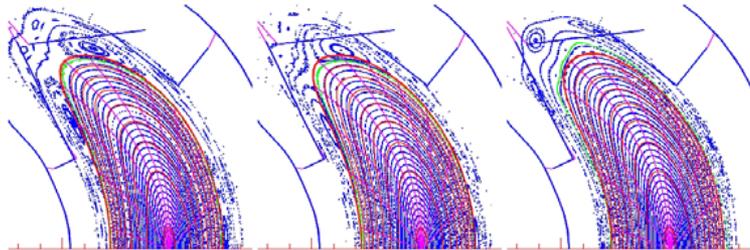
- Island width should be $\propto I_{trim}^{1/2}$
- Data: small residual island may exist, consistent with estimated construction errors.

S. Lazerson *et al.*, APS-DPP Meeting, Nov. 2015

3D Equilibrium Analysis Will Enable Diagnosis, Understanding, and Control



3D equilibrium model of the W7-X plasma obtained with **V3FIT**.

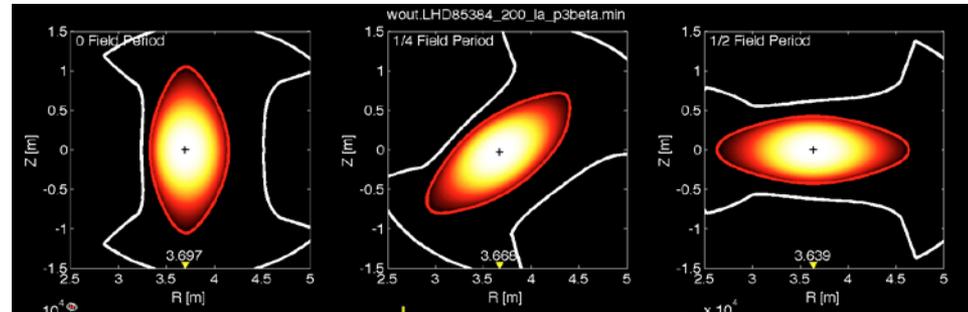


VMEC and w7-extender surfaces with varying bootstrap current

Measurement data: magnetics, profiles, coil currents

Reconstructed 3D equilibrium

Chordal data inversion, e.g. XICS
Physics analysis, e.g. EMC3-EIRENE

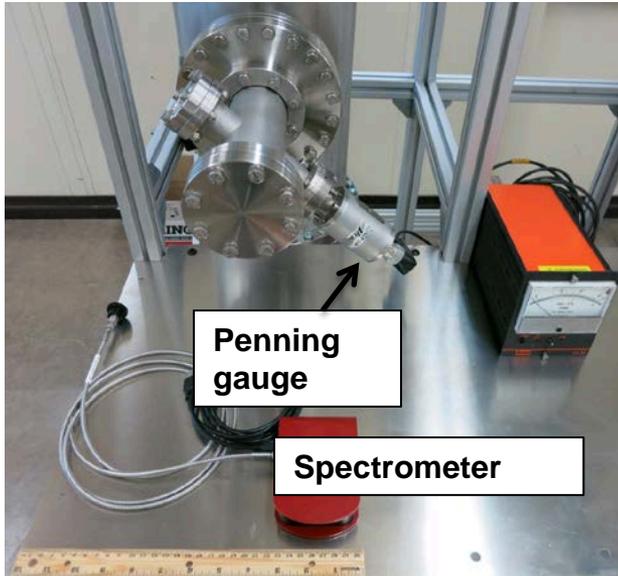


LHD equilibrium reconstruction using **STELLOPT** and **DIAGNO**

Carrying out the U.S. Agenda...

Divertor-related understanding and control

U.S. Edge / PMI Research Tools Are Installed and Getting Data

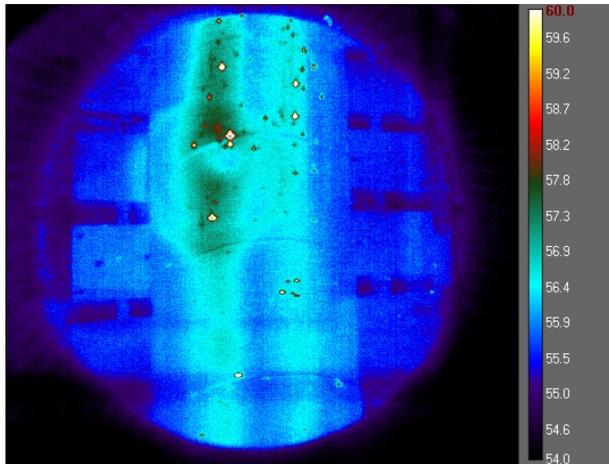
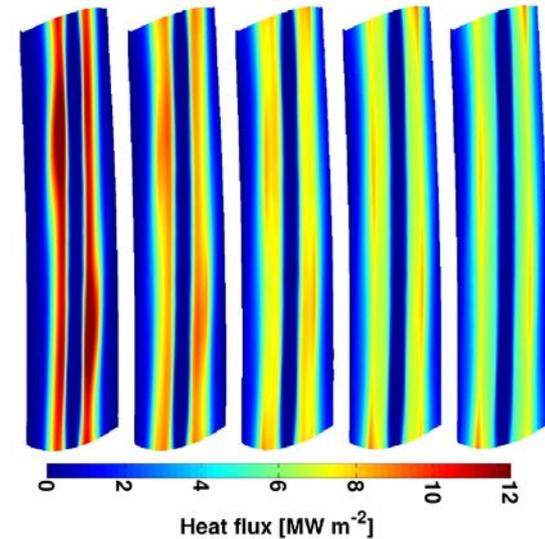


High resolution neutral gas analyzer



Scaling of limiter heat fluxes

$1.0 \times 10^{19} \text{ m}^{-3} \rightarrow$ Density scan $\rightarrow 8.0 \times 10^{19} \text{ m}^{-3}$



High resolution Infrared camera



Edge transport and PMI modeling predictions with EMC3-EIRENE

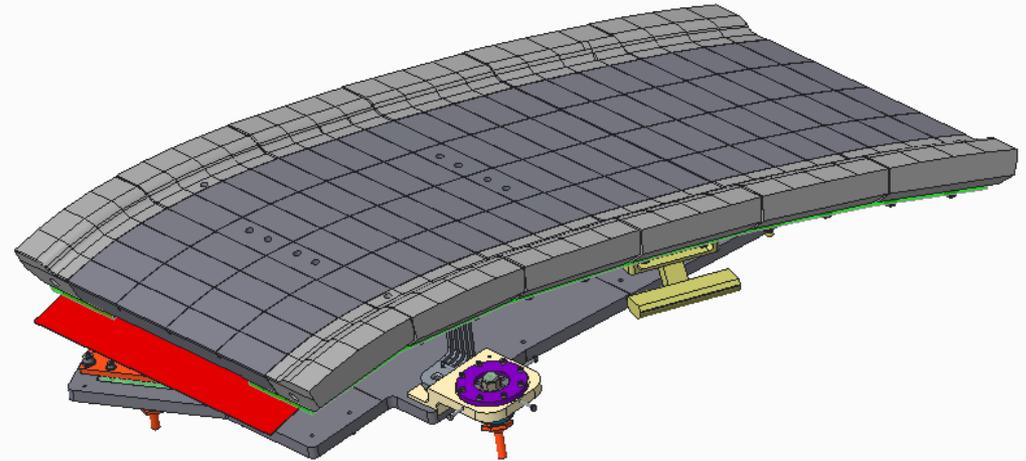
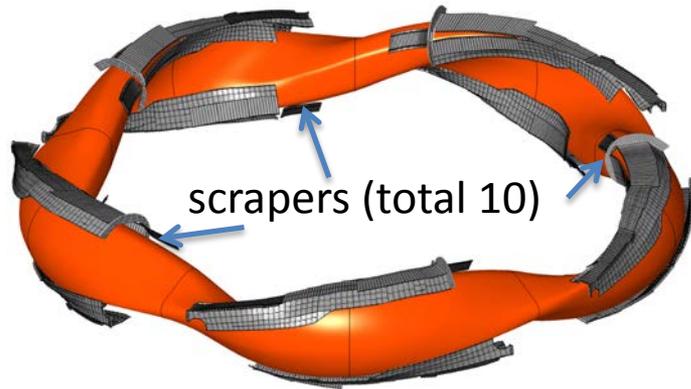


and... Edge plasma spectroscopy filterscopes



Instrumented Divertor “Scraper” will enable edge plasma and PMI control studies

$n=5/m=5$ island divertor



Design / fab. by ORNL / PPPL / IPP team. Delivery in Sept. 2016

| | U.S. | IPP |
|--------------------|------|-----|
| Thermocouples | | ✓ |
| Langmuir Probe | ✓ | ✓ |
| Plenum pressure | | ✓ |
| Visible cameras | | ✓ |
| Infrared cameras | ✓ | ✓ |
| Magnetics modeling | ✓ | ✓ |
| Plasma modeling | ✓ | ✓ |

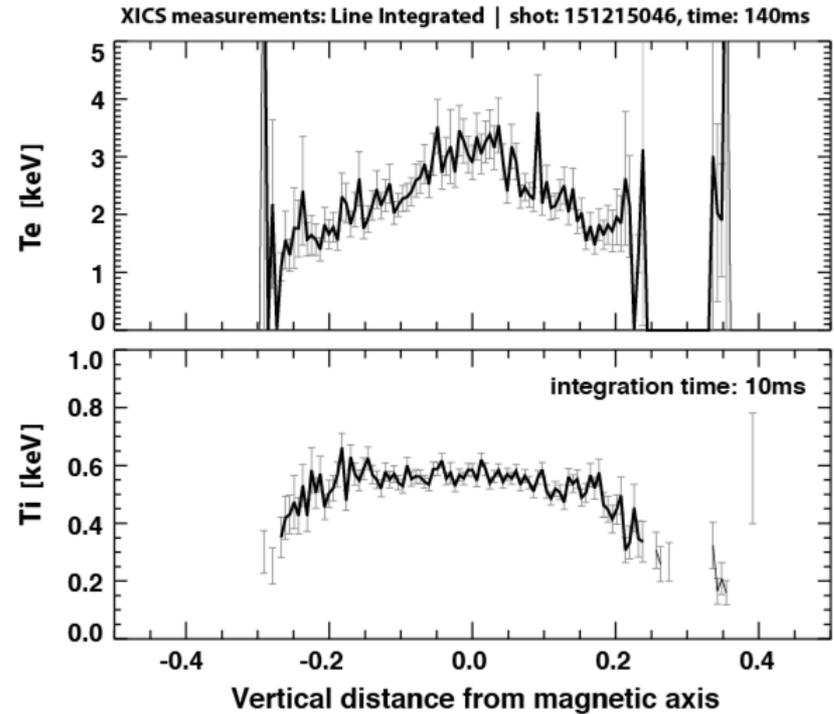
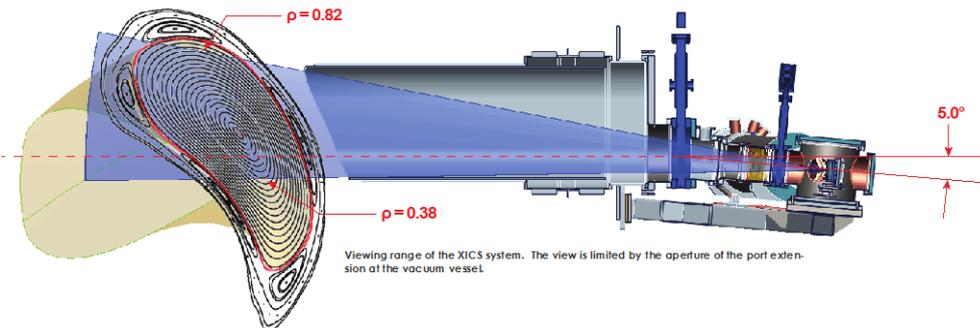
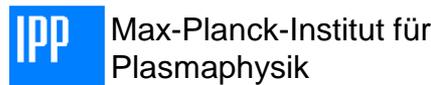
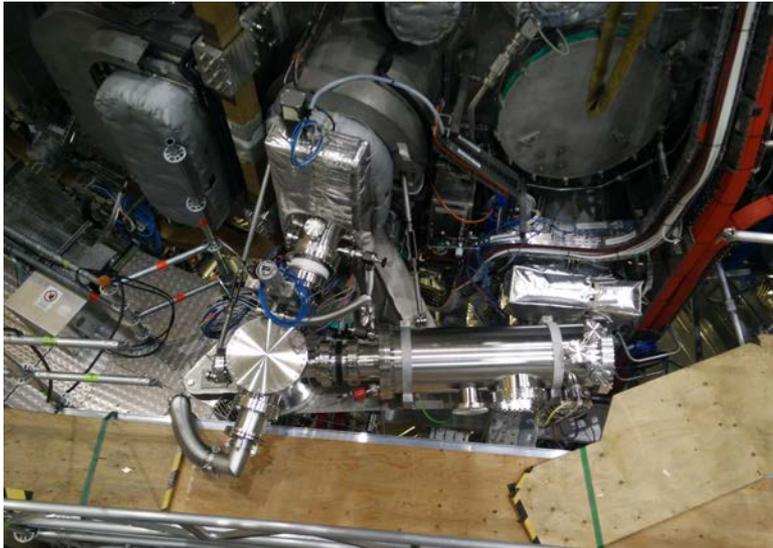
- Will the scraper protect the divertor target edges as predicted?
- How will divertor pumping be affected?

Carrying out the U.S. Agenda...

What governs transport?

First Temperature Profile Measurements! (Preliminary)

X-ray Imaging Crystal Spectrometer



Will measure profiles of:

- Ion temperature
- Electron temperature
- Plasma flow
- Impurity density

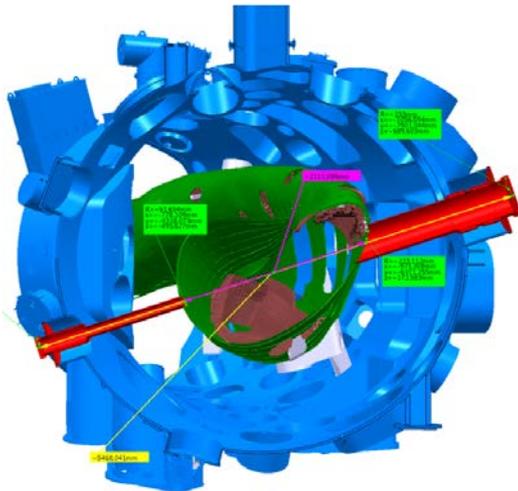
Fluctuation Diagnostics in Preparation

Phase Contrast Imaging

- Measure turbulent fluctuations, e.g., ITG, TEM and ETG with frequencies ≤ 1 MHz and $k \leq 30$ cm⁻¹.
- Currently used on Alcator C-Mod, DIII-D, LHD, TCV for edge and core fluctuations.



PCI beam geometry
in W7-X



Gas Puff Imaging



- Measure plasma edge turbulence using fast (~ 1 ms) 2D imaging of emission from an edge-localized gas puff.
- Also possible to study island divertor dynamics

Heavy Ion Beam Probe

- Simultaneously measure density and potential fluctuations, $E_r(r)$
- Use equipment developed and operated on TEXT-U.
- Feasibility study in progress.

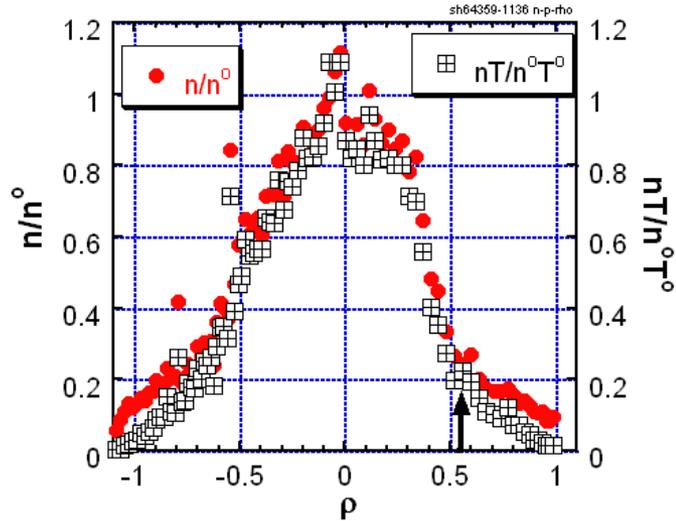


Xantho Technologies, LLC

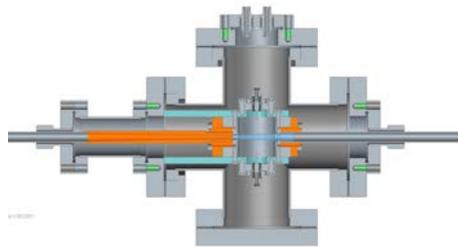
Carrying out the U.S. Agenda...

Density profile control

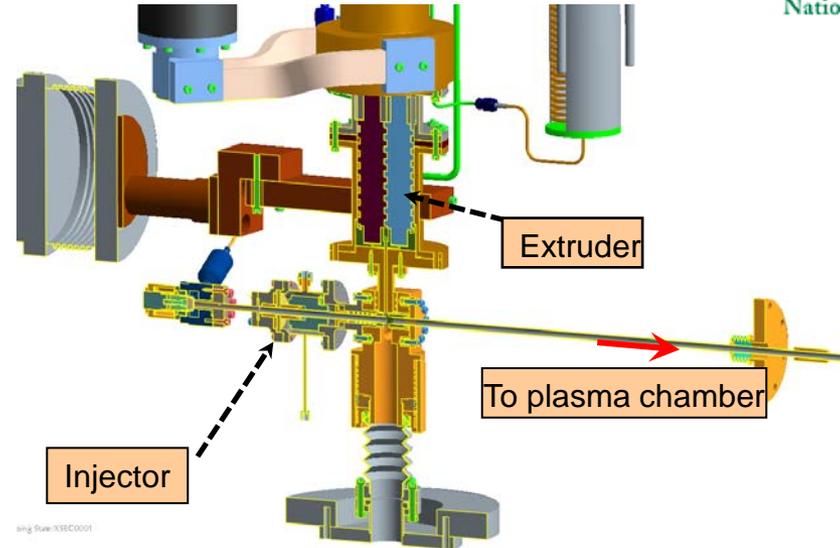
U.S.-IPP Collaboration in Pellet Injection for W7-X



LHD achieves peaked density profiles, best performance with pellets.



Phase 1: U.S. will supply pellet mass detectors for existing short-pulse injector.



Phase 2: Steady-state system combining U.S.-supplied frozen hydrogen extruder with IPP injector.

- Will provide steady-state core fueling and a field test of U.S.-developed ITER extruder technology.

MDSplus Data Acquisition & Management at W7-X

- W7X is now using MDSplus to support a subset (currently 7) of their diagnostics.
 - Diagnostics from outside collaborators (MIT, PPPL, CIEMAT) are being delivered on-site with MDSplus
 - All data stored by the native CODAC system is also available through MDSplus.
- Several more diagnostics (10-20) will be added to MDSplus for the next campaign.
 - This is a temporary solution while W7-X CODAC development is continuing.
- W7-X has adopted MDSplus for remote data access. As at JET and AUG, this is advantageous for remote collaborators.
- US DOE supports this effort under general MDSplus funding at MIT.



W7-X Offers Unique Opportunities to Advance FES Research Priorities

Example: Plasma-Material Interactions (ref., PMI Workshop PRDs)

- Mid-term: Exploit U.S. investments in the divertor scraper, trim coils, and PMI-related diagnostics.
 - Tests the innovative island divertor concept (PRD 1)
 - Compatibility with an optimized core (PRD 5)
 - Test trim coils as an actuator for controlling loading uniformity among the 10 divertors (PRD 1)
- Long-term: Extend in-situ PMI material capabilities developed on U.S. machines to long-pulse. (PRD 1, 3)
 - Flexible sample exposure techniques, e.g., DIMES.
 - In-vacuo post-exposure surface analysis, e.g. MAPP probe.
 - Large wall area material evolution, e.g. accelerator based in-situ material surveillance (AIMS)

**W7-X doors are wide open to U.S. scientists.
Opportunities are only limited by our ability to participate.**

Toward a Scientifically Productive Collaboration on an Overseas Facility

Lessons learned from what has worked and what hasn't...

Some requirements:

1. U.S. contributions to essential capabilities, and involvement from Day 1. ✓
2. Research staffing sufficient to extract the science from our hardware contributions.
 - Dedicated core team to ensure key U.S. leadership roles, first-author science publications, visible representation at conferences.
3. A strong, permanent on-site team for effective integration with the host team, other partners, and program planning.
 - Currently an open need.
4. An effective remote collaboration model, designed to optimize U.S. scientific productivity.
 - BPO model for ITER* has excellent ideas adaptable for W7-X.
 - Survey of W7-X collaborators (and many others) is in progress, soliciting input needed to set requirements.

* M. Greenwald, *et al.*, "Recommendations for ITER Experimental Operation, U.S. Team Formation and Participation," U.S. BPO Report, April 2015.

Summary

- A scientifically promising U.S. collaboration with W7-X is off to an excellent start.
- The program is rich in opportunities to advance U.S. science interests. The doors are wide open to us.
- Transition from equipment preparation to scientific exploitation is challenging, but successful precedents exist.

W7-X web site: <http://www.ipp.mpg.de/16900/w7x>

U.S. W7-X information: <http://advprojects.pppl.gov/home/w7-x>

Backup

Summary of Current U.S. Team Participation in W7-X

Massachusetts Institute of Technology

- Phase contrast imaging, gas puff imaging, MDS-plus

University of Wisconsin

- Penning gauge neutral gas analyzer, edge transport / PMI simulation

Auburn University

- Equilibrium analysis, x-ray imaging crystal spectrometer (w/PPPL).

Xantho Technologies

- Heavy ion beam probe feasibility study.

Oak Ridge National Laboratory

- Divertor scraper design, edge filterscopes.

Los Alamos National Laboratory

- Infrared imaging

Princeton Plasma Physics Laboratory

- Field mapping experiments, magnetic configuration control (trim coils), high-resolution temperature/velocity profiles (x-ray imaging crystal spectrometer), divertor scraper design (w/ORNL) and fabrication, national coordination



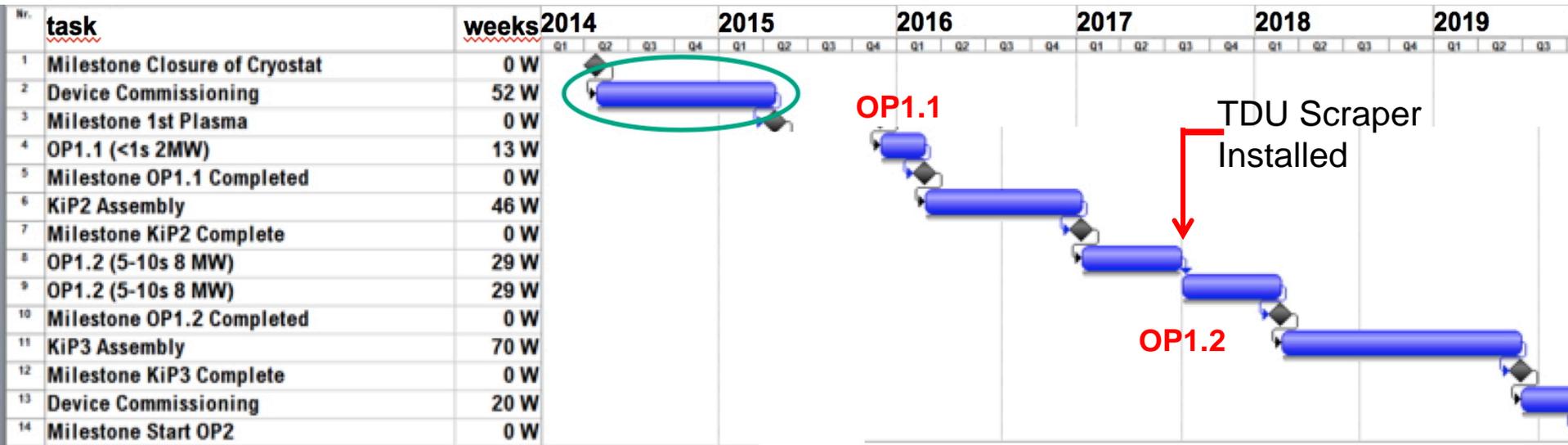
Near-term planning „updated“



device
commissioning

test divertor
assembly

HHF divertor
assembly



plasma commissioning
diagnostics/control
first investigations

1st plasma
w/o divertor

1st divertor
plasmas

steady-state
plasmas 2020

