

FUSION, Future Vision of Green Energy

Fusion Energy Research and Development in “ITER Era”

- A Korean Perspective

Gyung-Su LEE
National Fusion Research Institute



part

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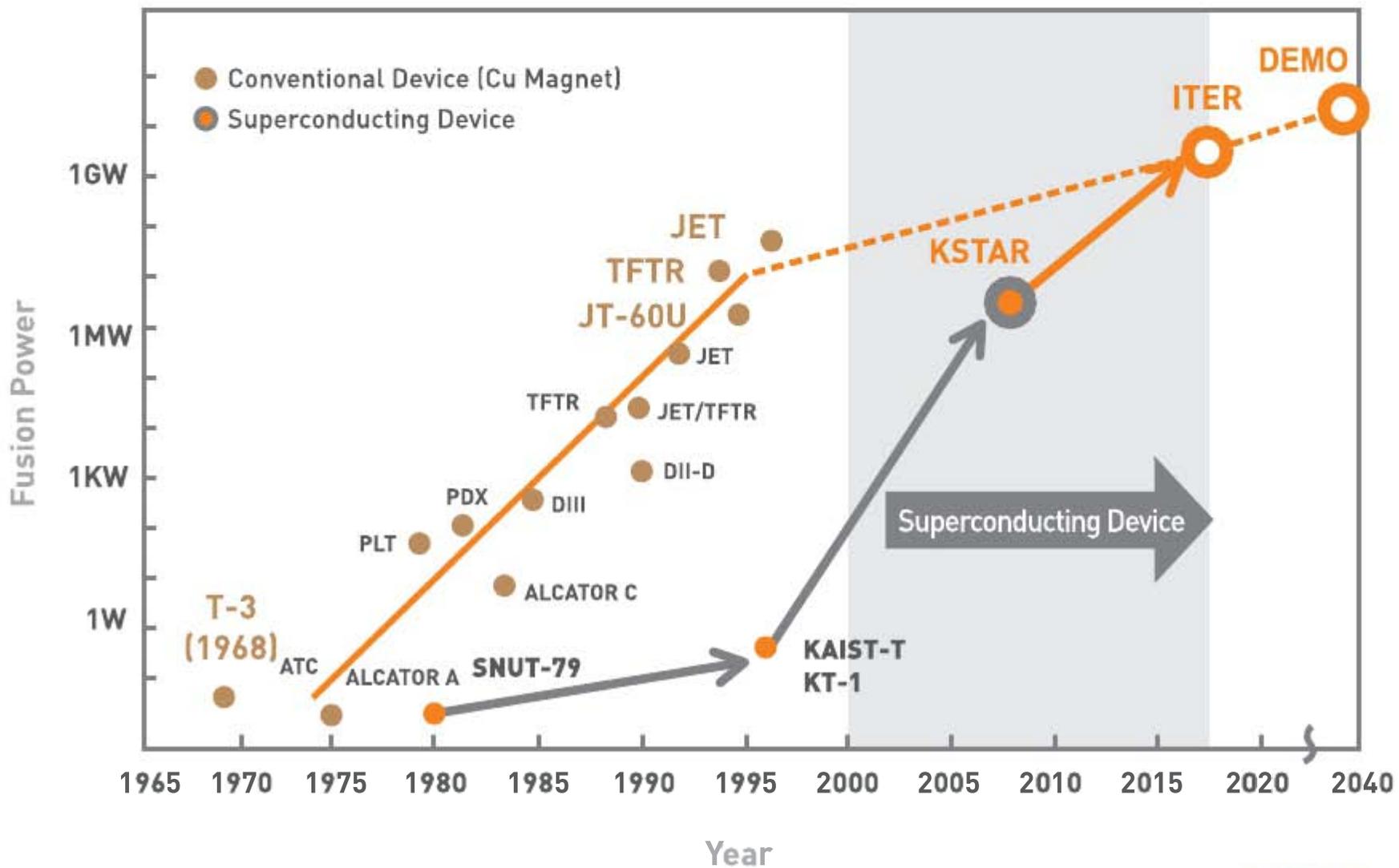
National Fusion Research Institute

Past and Present : KSTAR

NFRI 국가핵융합연구소
National Fusion Research institute



Initiation of KSTAR Project based on “Mid-Entry Strategy”





Establishment of National Fusion Research Center



Opening Ceremony of National Fusion Research Center in January 1996

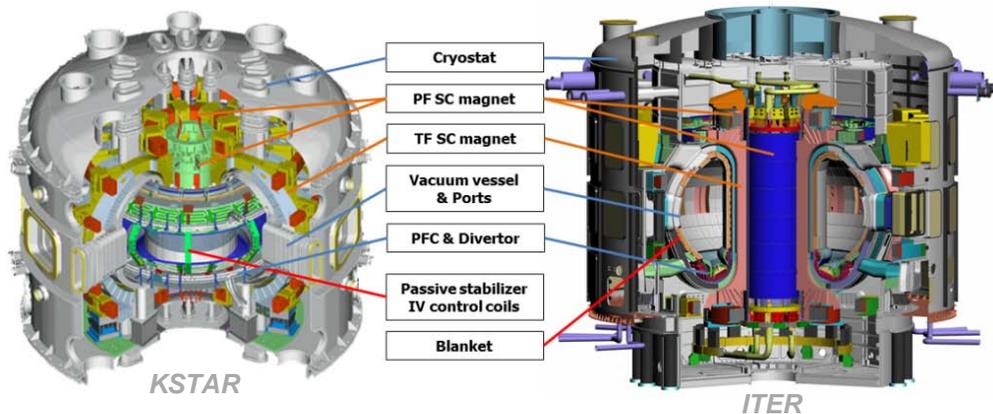




KSTAR Project Mission and Parameters

❖ KSTAR Mission

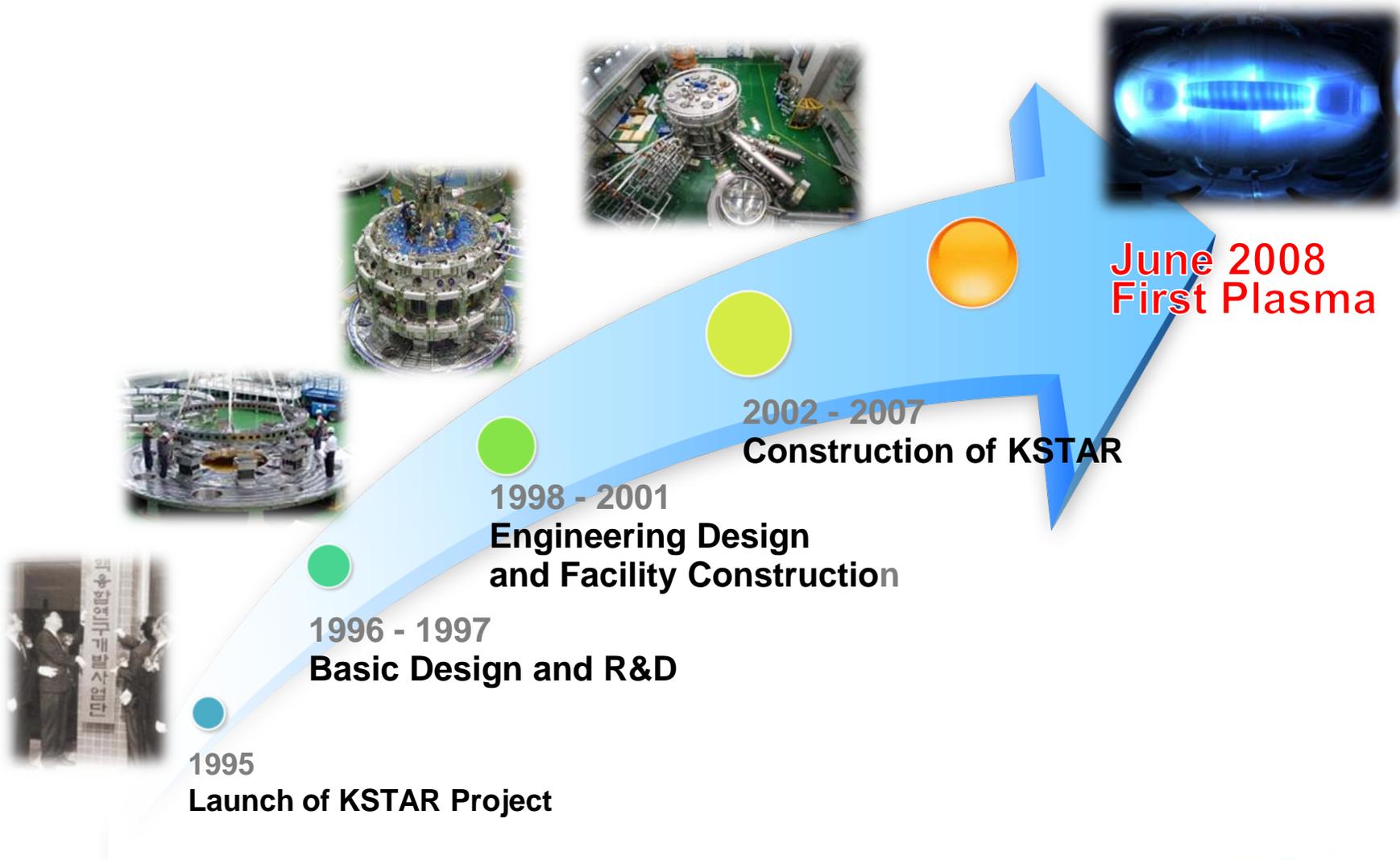
- To achieve the superconducting tokamak construction and operation experiences
- To develop **high performance steady-state operation physics and technologies** that are essential for ITER and fusion reactor development



PARAMETERS	Designed	2010 Op
Major radius, R_0	1.8 m	1.8 m
Minor radius, a	0.5 m	0.5 m
Elongation, κ	2.0	2.0
Triangularity, δ	0.8	0.8
Plasma volume	17.8 m ³	17.8 m ³
Bootstrap Current, f_{bs}	> 0.7	-
PFC Materials	C, CFC (W)	C
Plasma shape	DN, SN	DN, SN
Plasma current, I_p	2.0 MA	0.5 MA
Toroidal field, B_0	3.5 T	3.5 T
Pulse length	300 s	5 s
β_N	5.0	1.0
Plasma fuel	H, D	D
Superconductor	Nb ₃ Sn, NbTi	Nb ₃ Sn, NbTi
Auxiliary heating /CD	~ 28 MW	2.0 MW



Technology Development of entire process of KSTAR from Concept Design to Commissioning!





Success of KSTAR is based on dedication and Technological Advancements of Korean Industries!

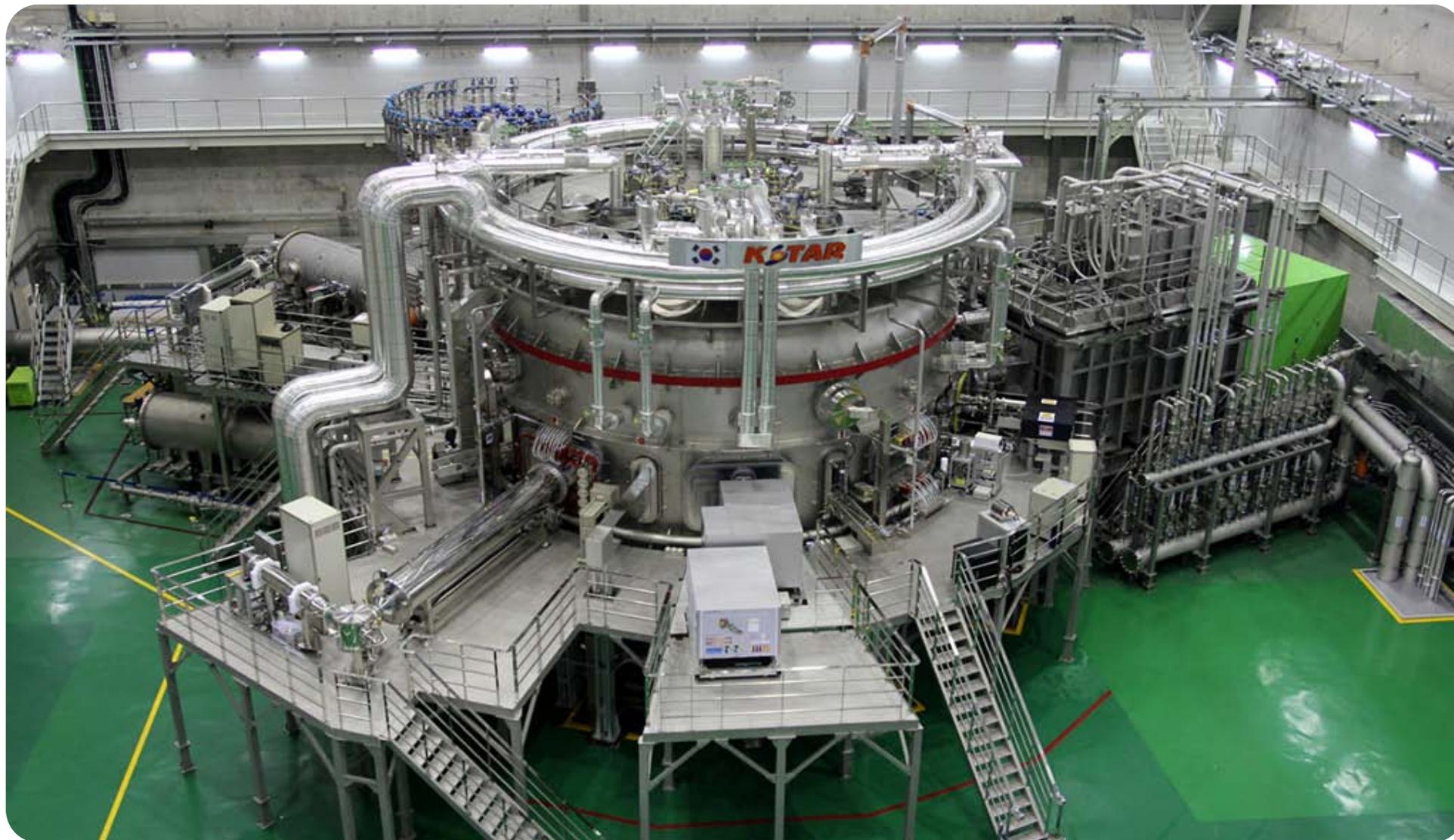


▶ 1,510 Participants
from 69 Companies



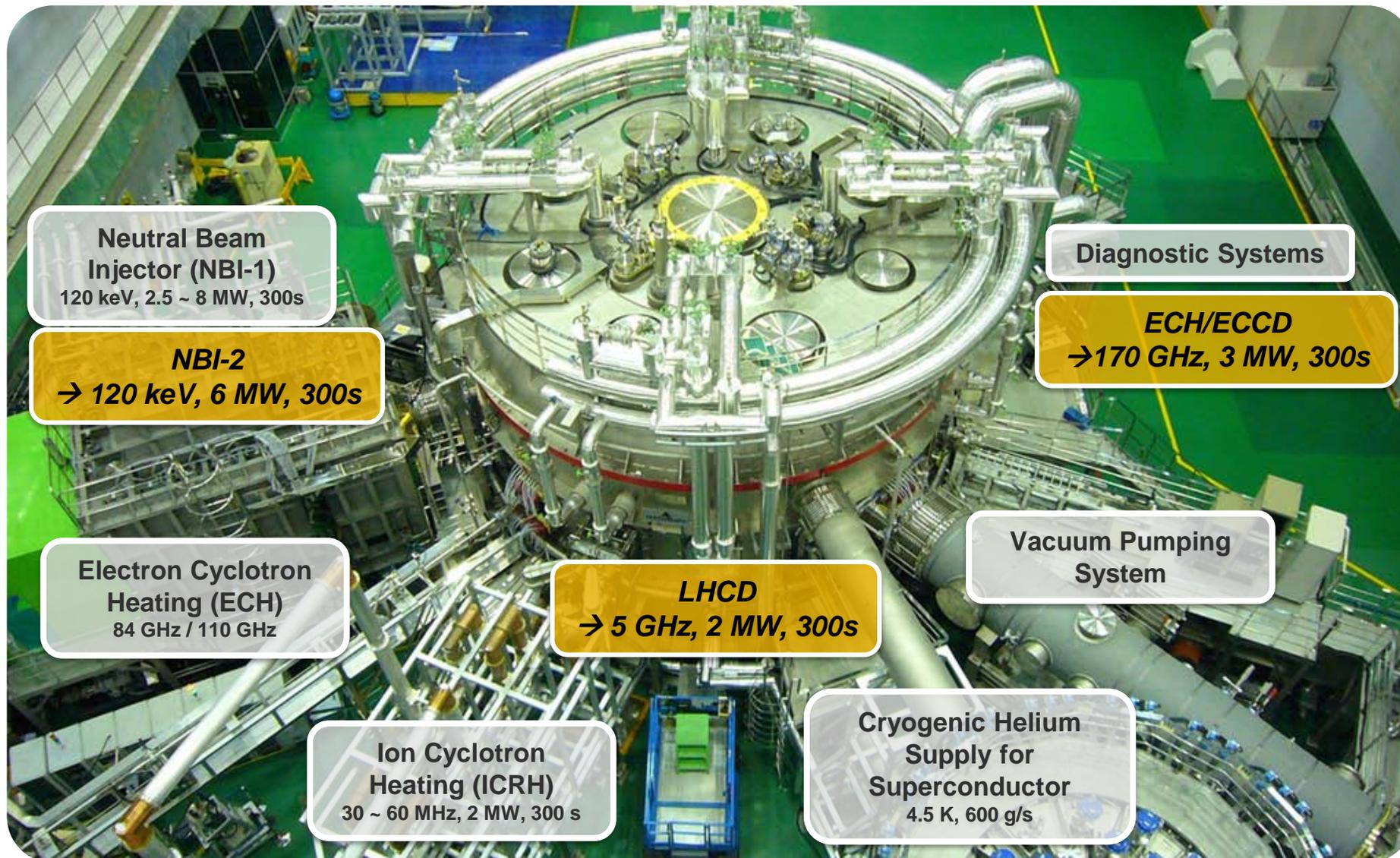
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Panoramic View of KSTAR Experimental Hall





Ancillary Systems of KSTAR in 2010 & Upgrade



**Neutral Beam
Injector (NBI-1)**
120 keV, 2.5 ~ 8 MW, 300s

NBI-2
→ 120 keV, 6 MW, 300s

**Electron Cyclotron
Heating (ECH)**
84 GHz / 110 GHz

**Ion Cyclotron
Heating (ICRH)**
30 ~ 60 MHz, 2 MW, 300 s

LHCD
→ 5 GHz, 2 MW, 300s

Diagnostic Systems

ECH/ECCD
→ 170 GHz, 3 MW, 300s

**Vacuum Pumping
System**

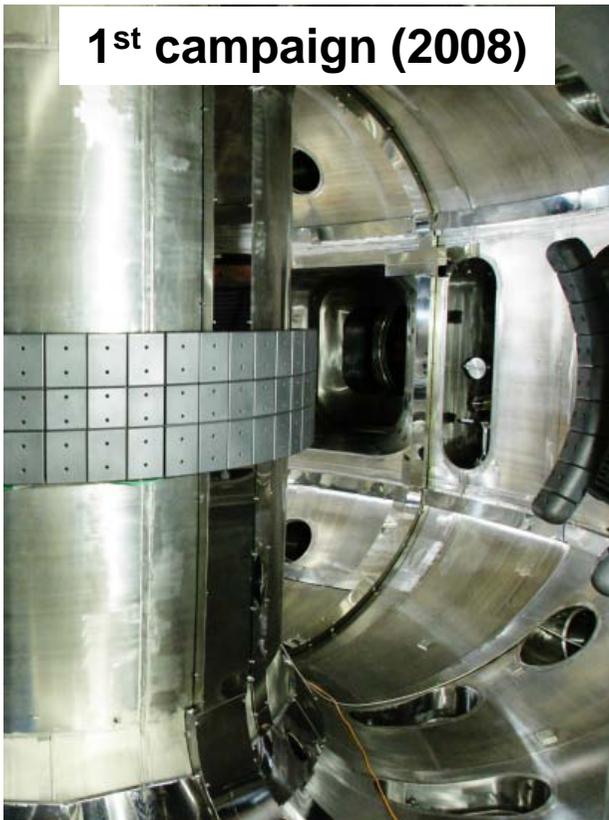
**Cryogenic Helium
Supply for
Superconductor**
4.5 K, 600 g/s



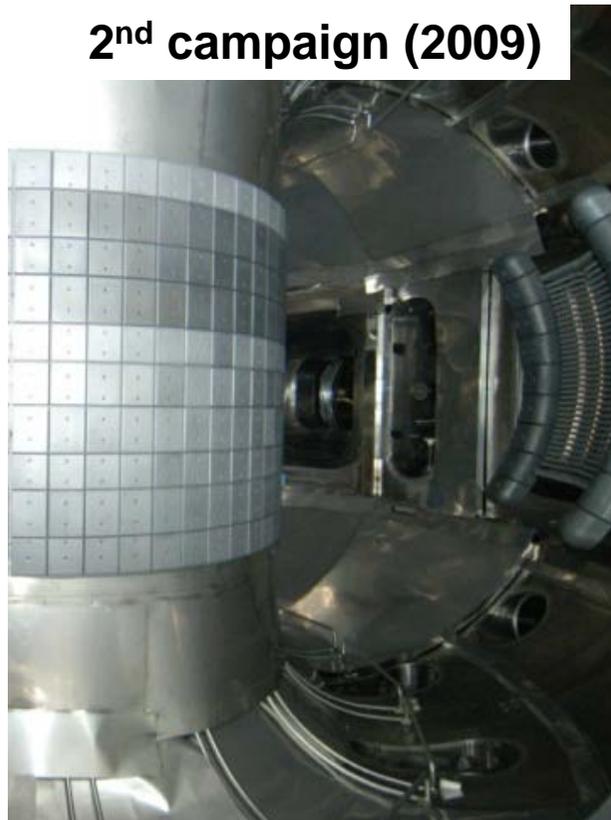
KSTAR In-Vessel System Upgrade

Whole in-vessel components were installed in 2010, including Divertors, In-vessel Coils, In-vessel Cryopumps, Passive Stabilizers, and In-board & Poloidal Limiters.

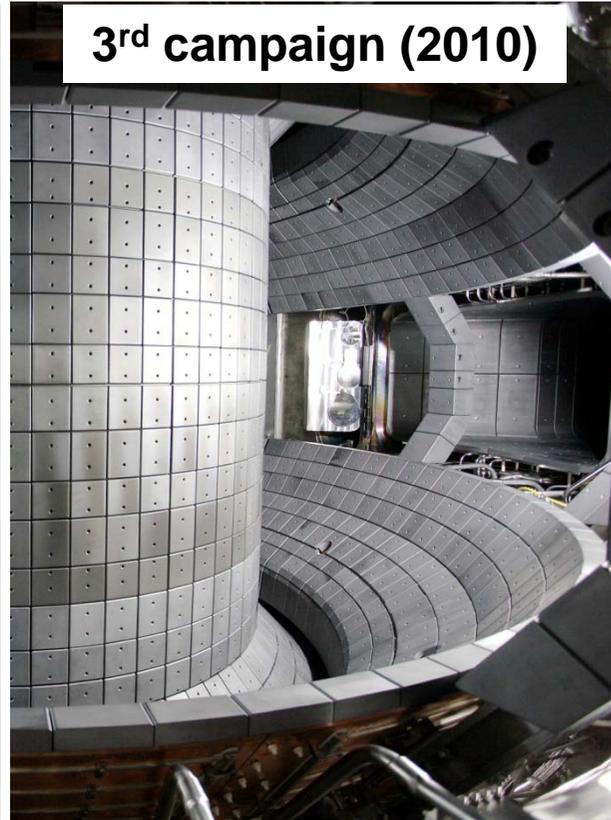
1st campaign (2008)



2nd campaign (2009)

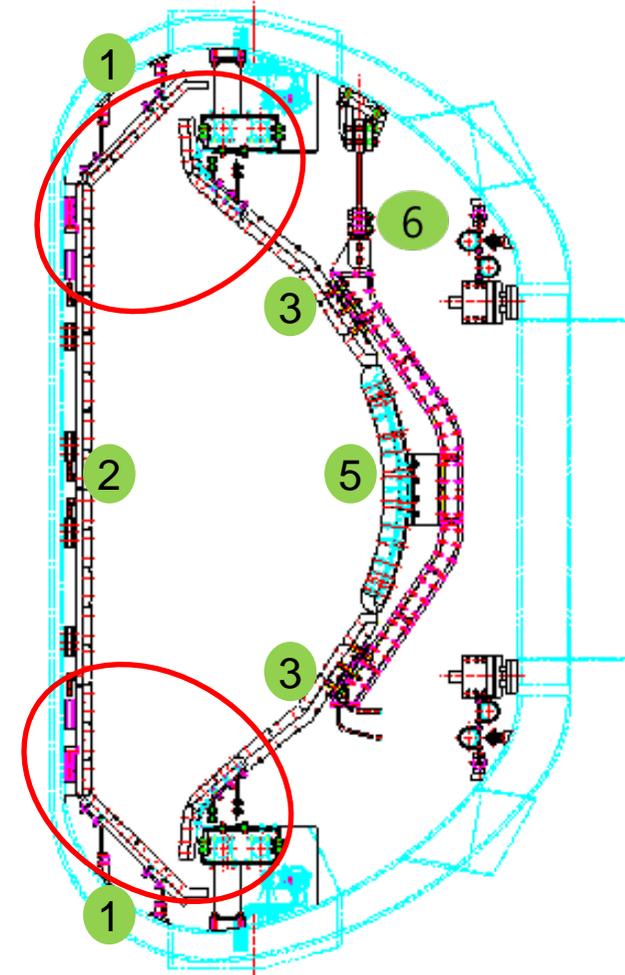
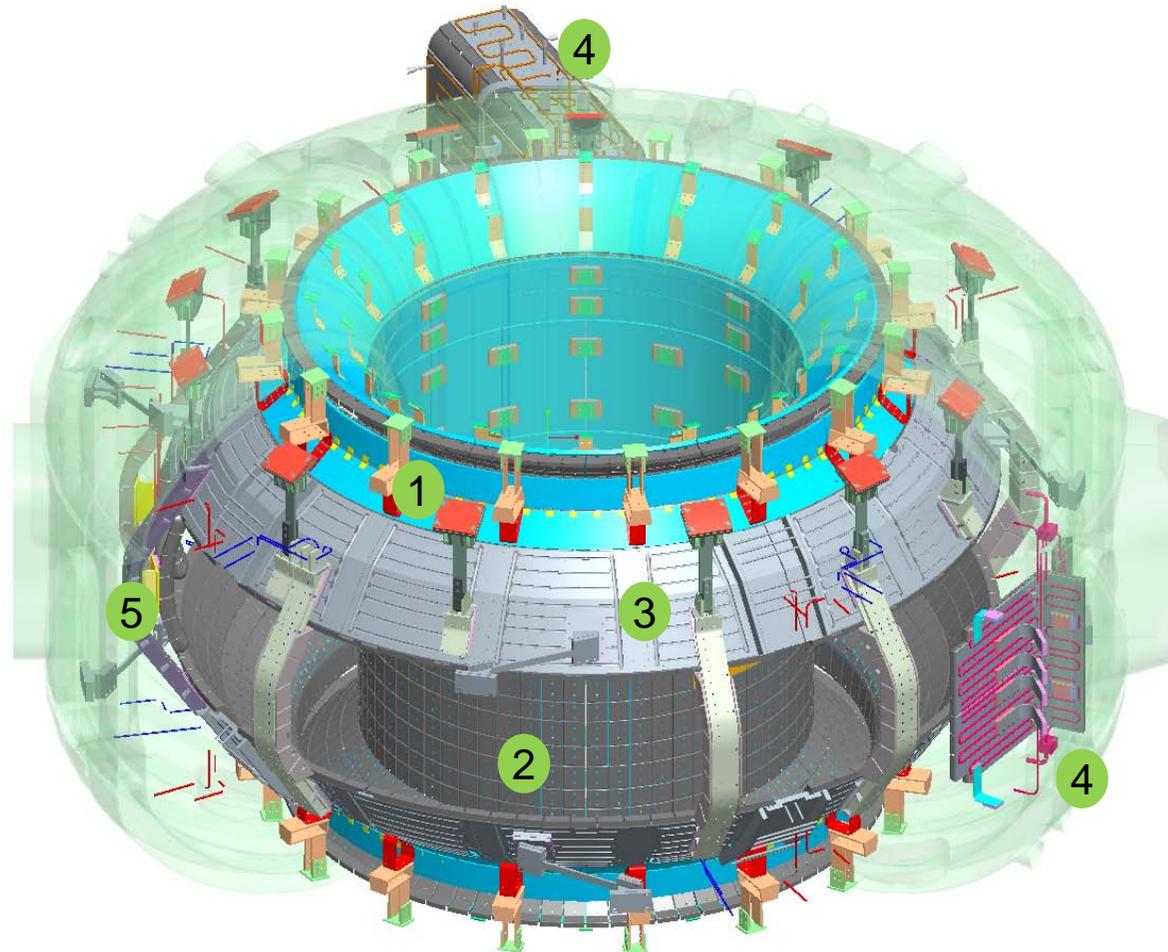


3rd campaign (2010)





KSTAR In-Vessel System Design



1 Divertor

2 Inboard Limiter

3 Passive Stabilizer

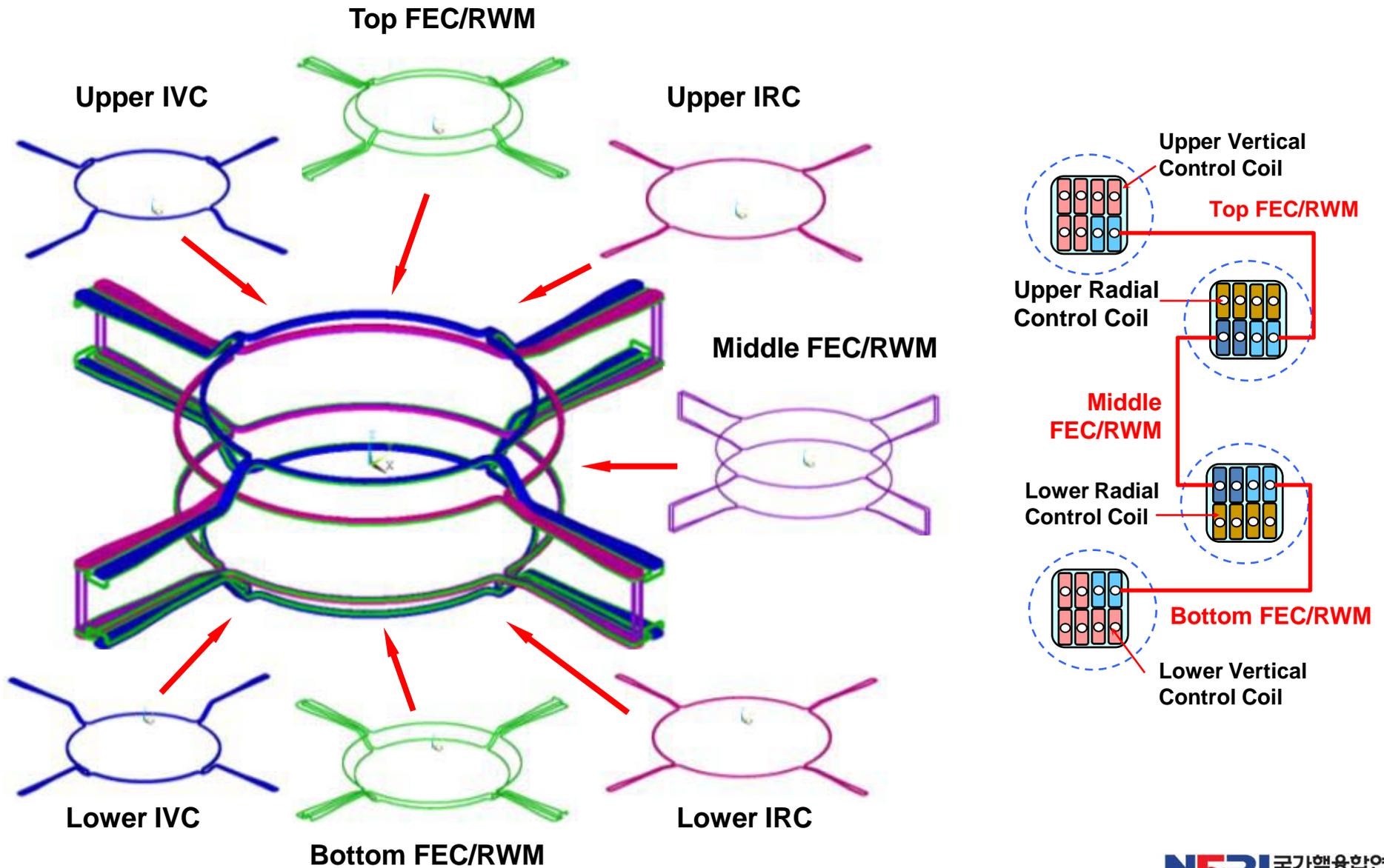
4 NB Armor

5 Poloidal Limiter

6 In-Vessel Control Coils



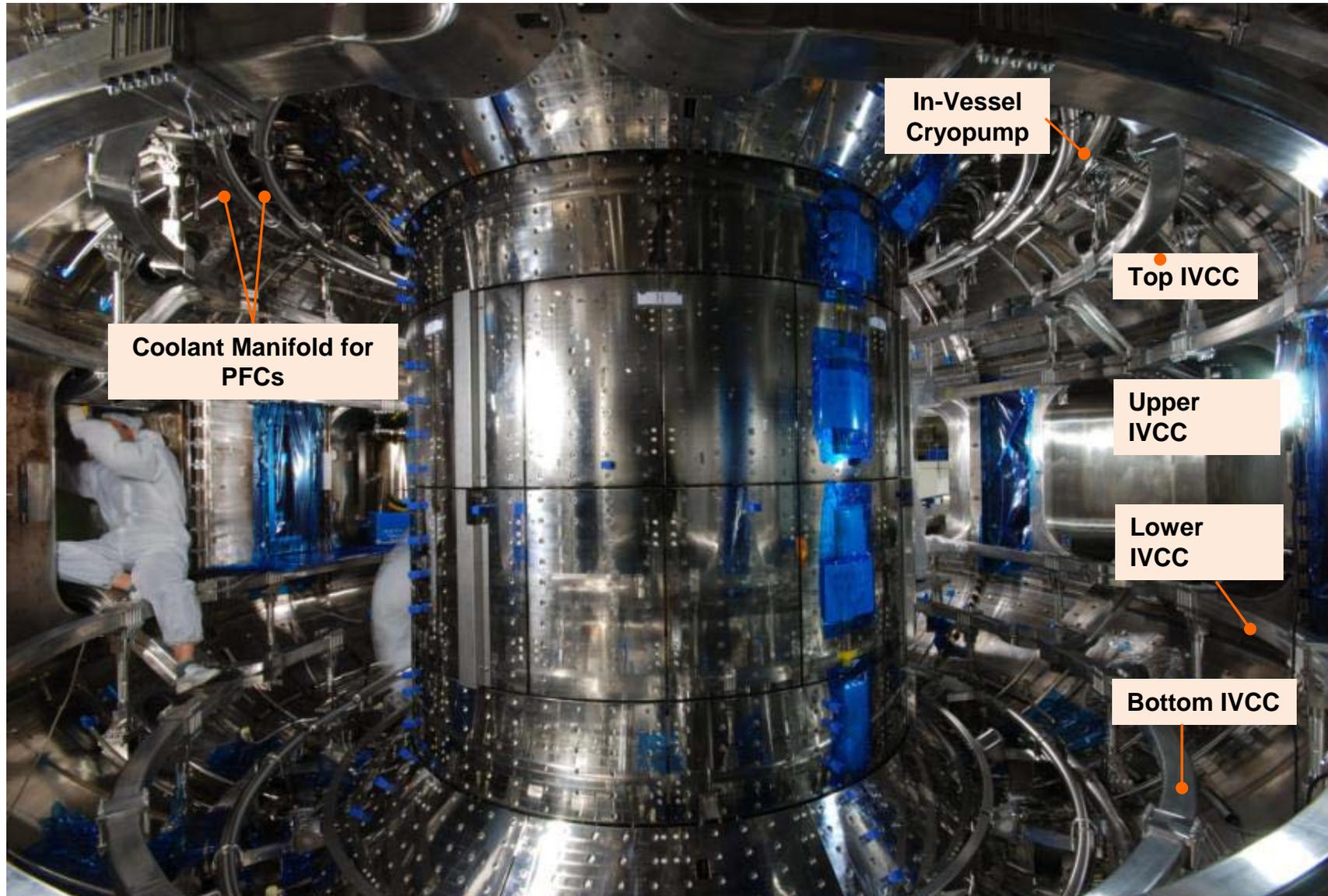
Segmented Coil Design of IVCC





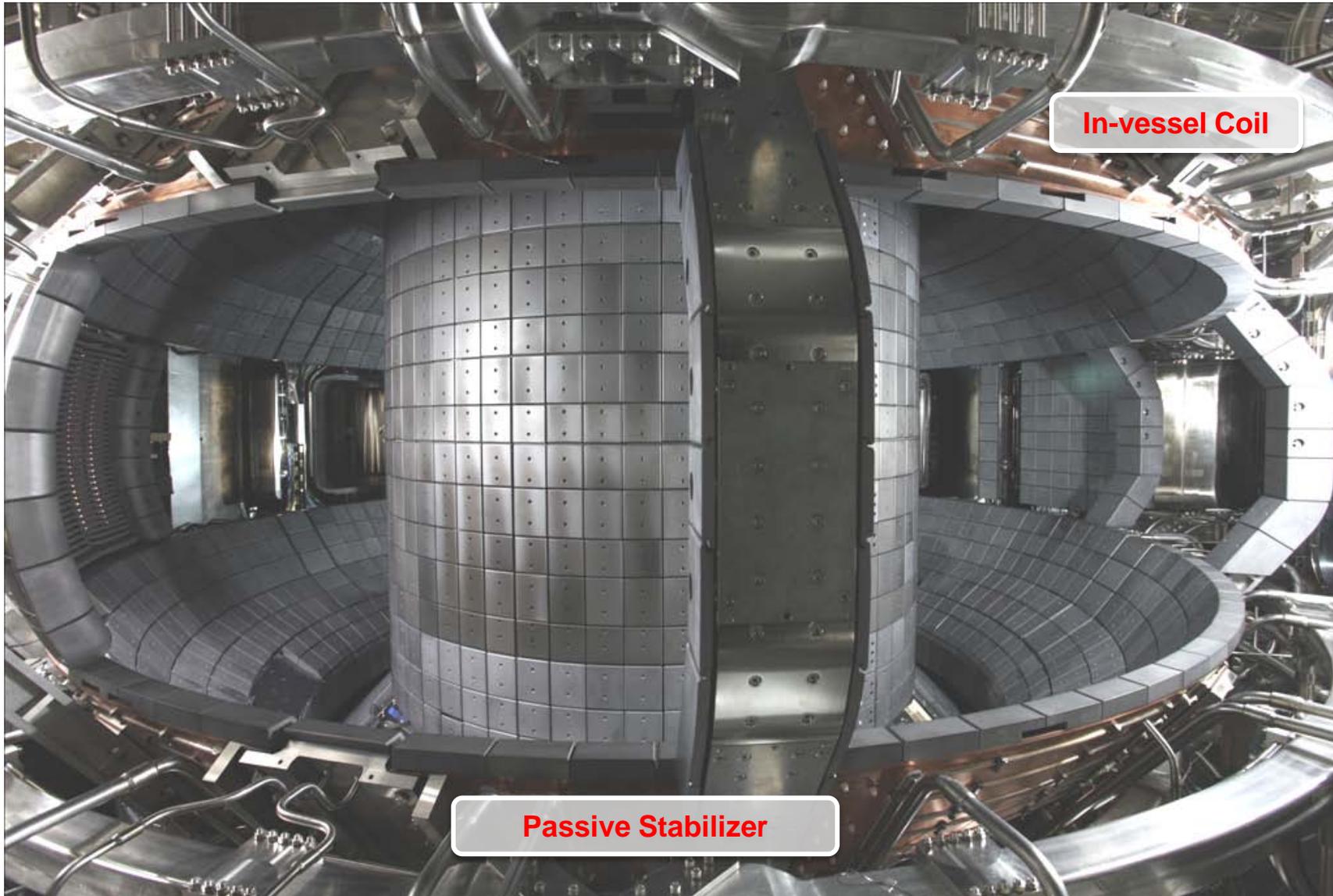
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KSTAR In-Vessel System Installation





KSTAR In-Vessel Systems in 2010 Campaign

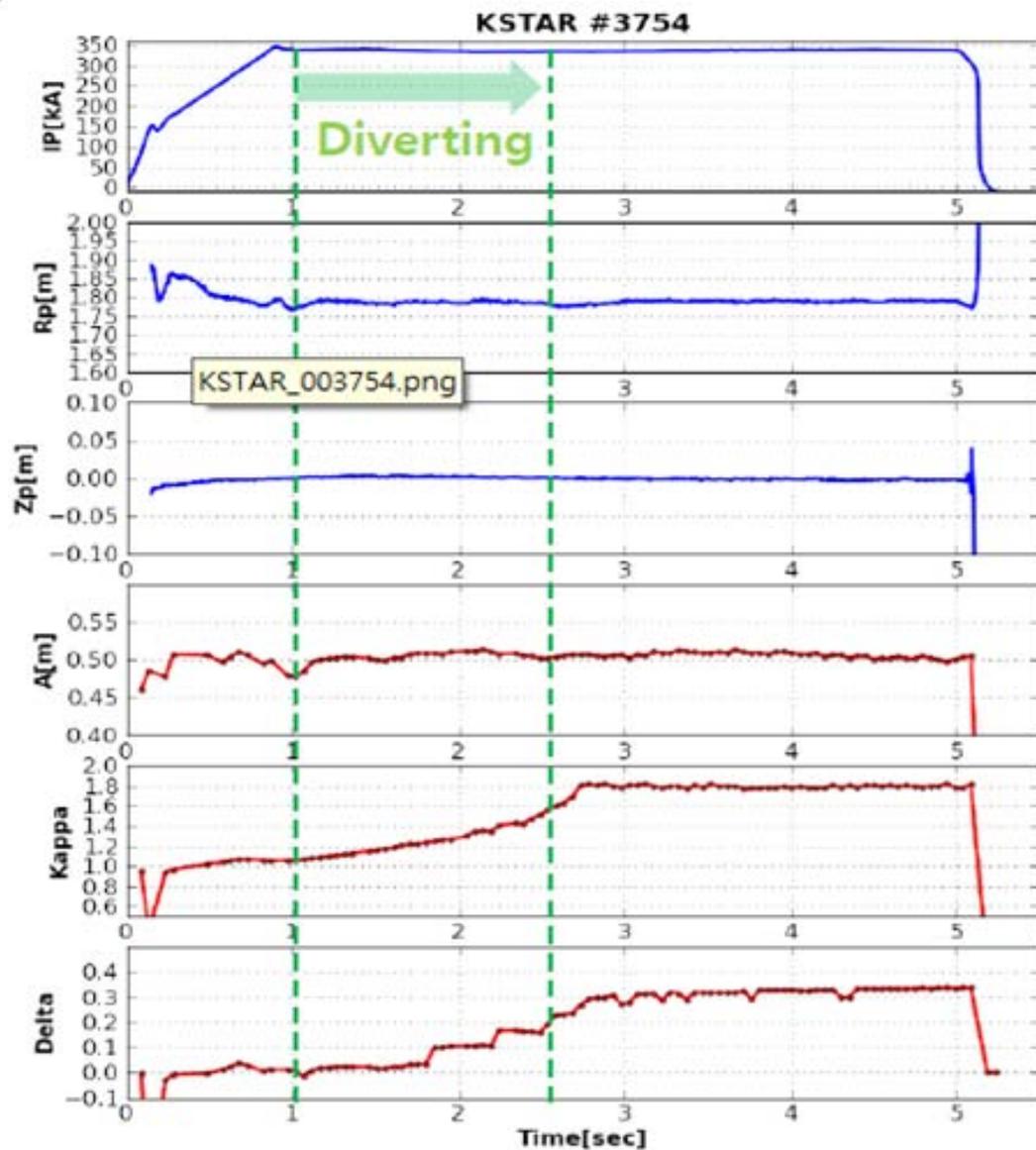


In-vessel Coil

Passive Stabilizer



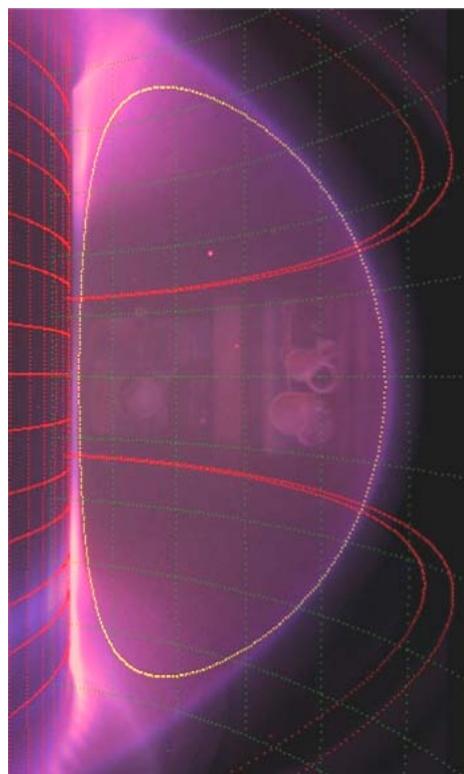
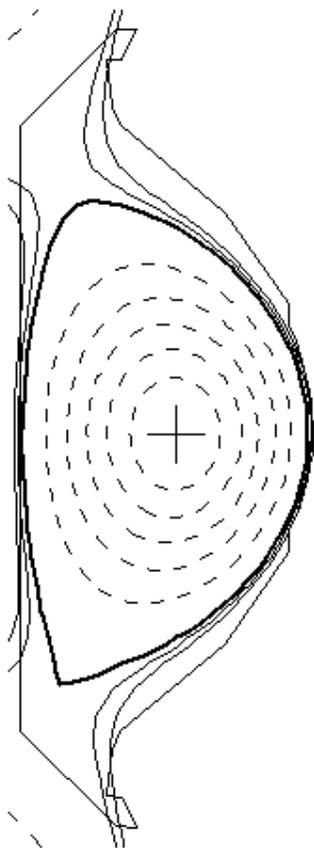
KSTAR D-shape Diverted Plasma





KSTAR Diverted H-mode Plasma

- **Shaping Capability : Plasma Shaping with vertical instability control**
- **Heating : 1.2 MW NBI, 0.4 MW 110GHz ECH**
- **Wall Conditioning with full carbon PFC : Baking @ 200 C ,**
- **Intensive He Glow Discharge Cleaning, Boronization with Carborane**

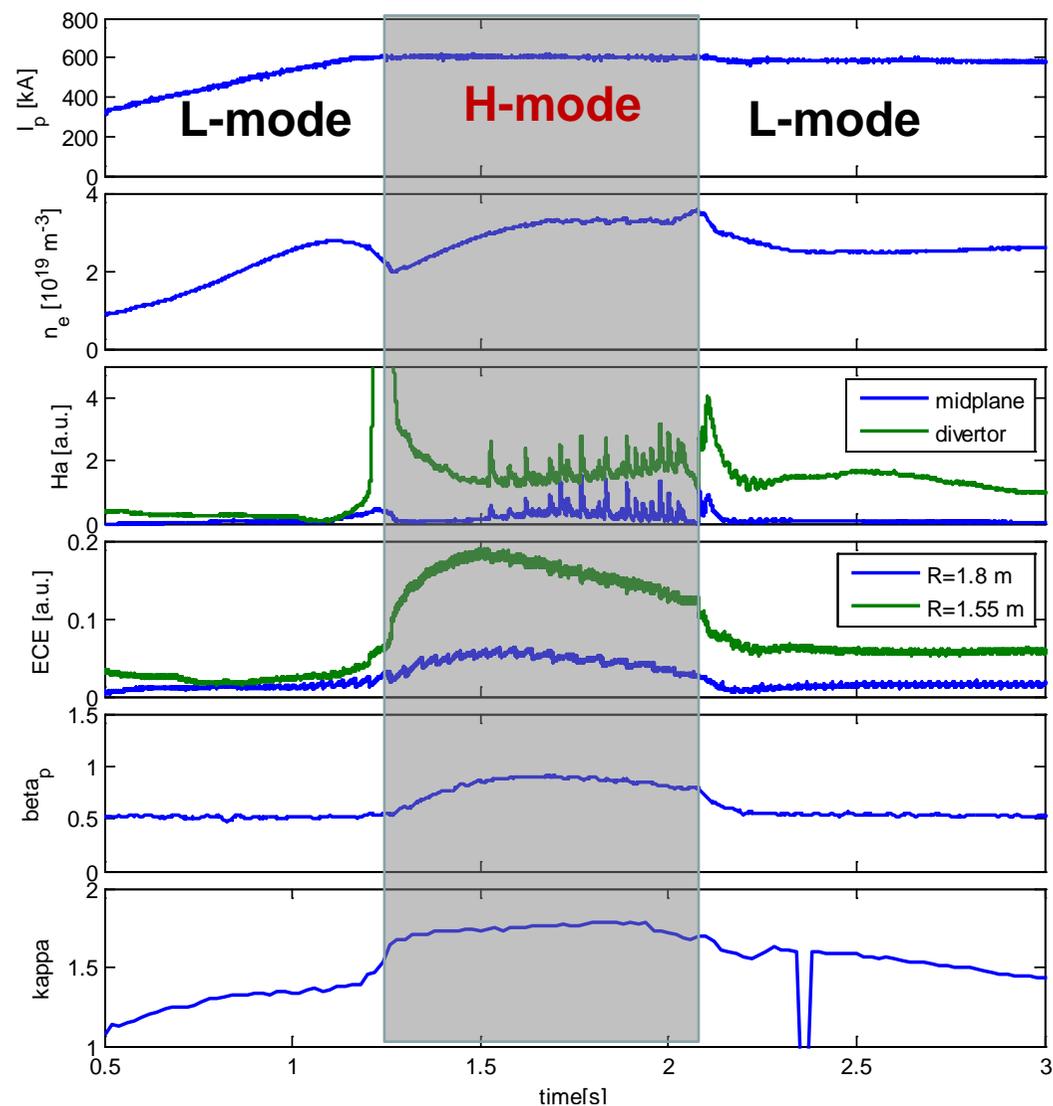


Shot #4333
 $\kappa \sim 1.7$
 $\delta \sim 0.4$
 $R_{gap} \sim 3\text{cm}$

- **Slightly shifted to the lower divertor**
- **Ion ∇B is at lower divertor**



Typical H-mode Discharge in KSTAR



Discharge #4333

$I_p \sim 0.6$ MA, $N_e \sim 2e19$ m⁻³

Double null, $\kappa \sim 1.8$

$B_T = 2$ T, $R \sim 1.8$ m, $a \sim 0.5$ m

Boronization with carborane

$P_{NBI} \sim 1$ MW (80 keV, co-NBI)

$P_{ECW} \sim 0.25$ MW (cntr - I_p)

$P_{OH} \sim 0.2$ MW

$P_{thres} \sim 1.1$ MW

(*ITER Physics Basis, 1999*)

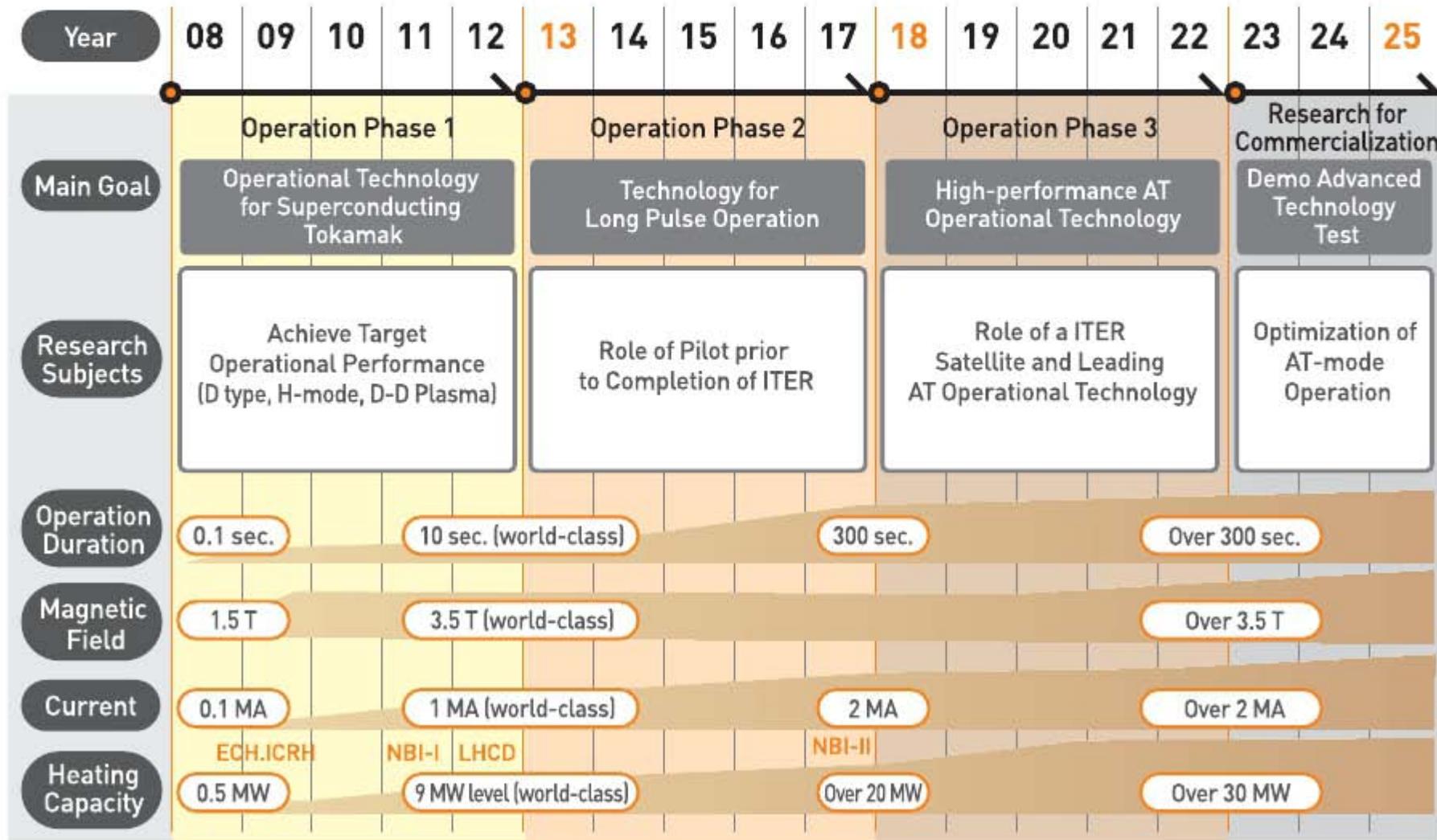
Type-I ELMs

sharp increase of edge ECE

80% increase of β_p

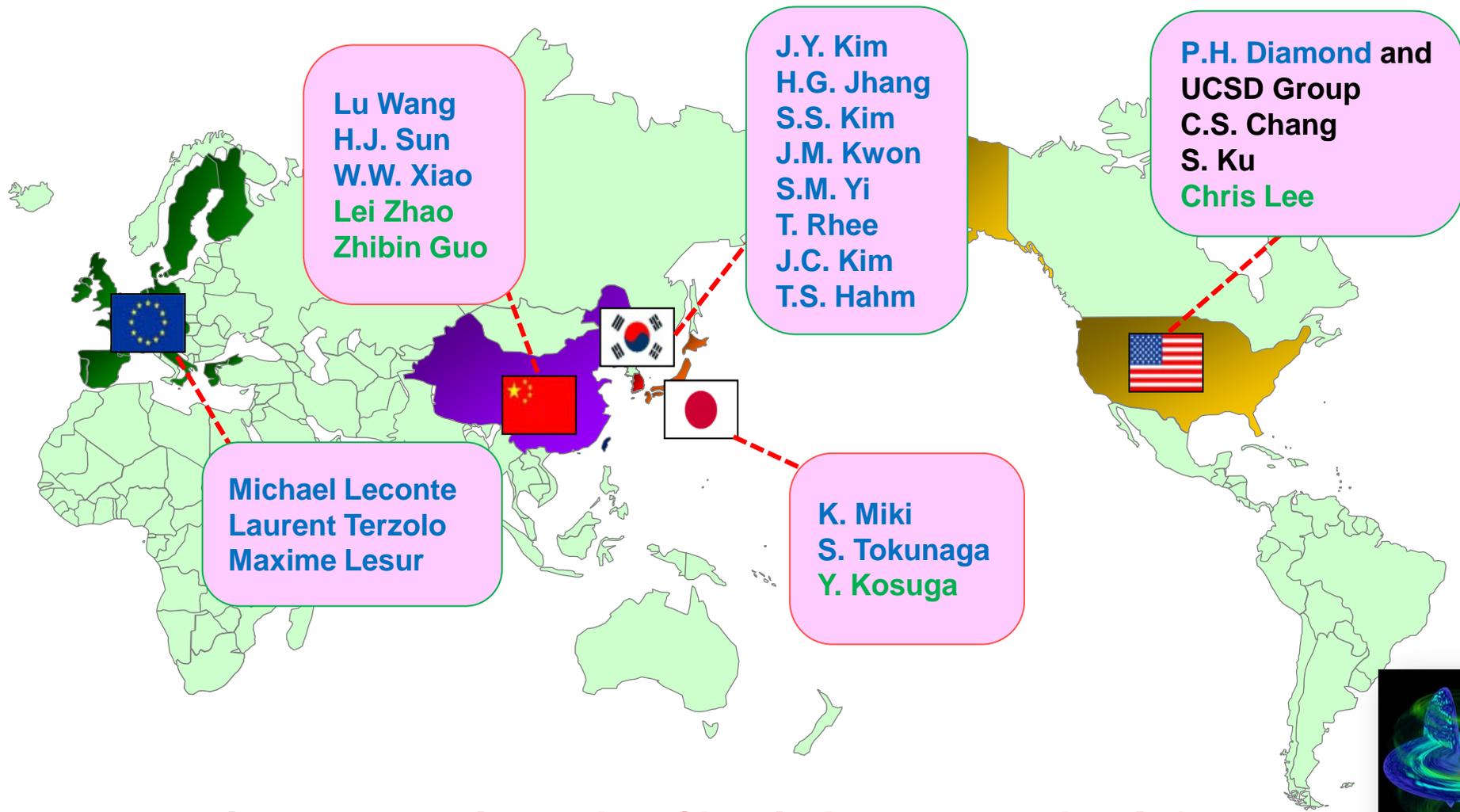


Phased Operational and Experimental Plans for KSTAR





WCI Center for Fusion Theory and Simulation



*** Goal : Large-scale Fusion Simulation and Fusion Informatics**



part

2



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Present and Future : ITER



ITER Facility Design and Construction Site



Birds-eye view of ITER site
Draft 02.03.09

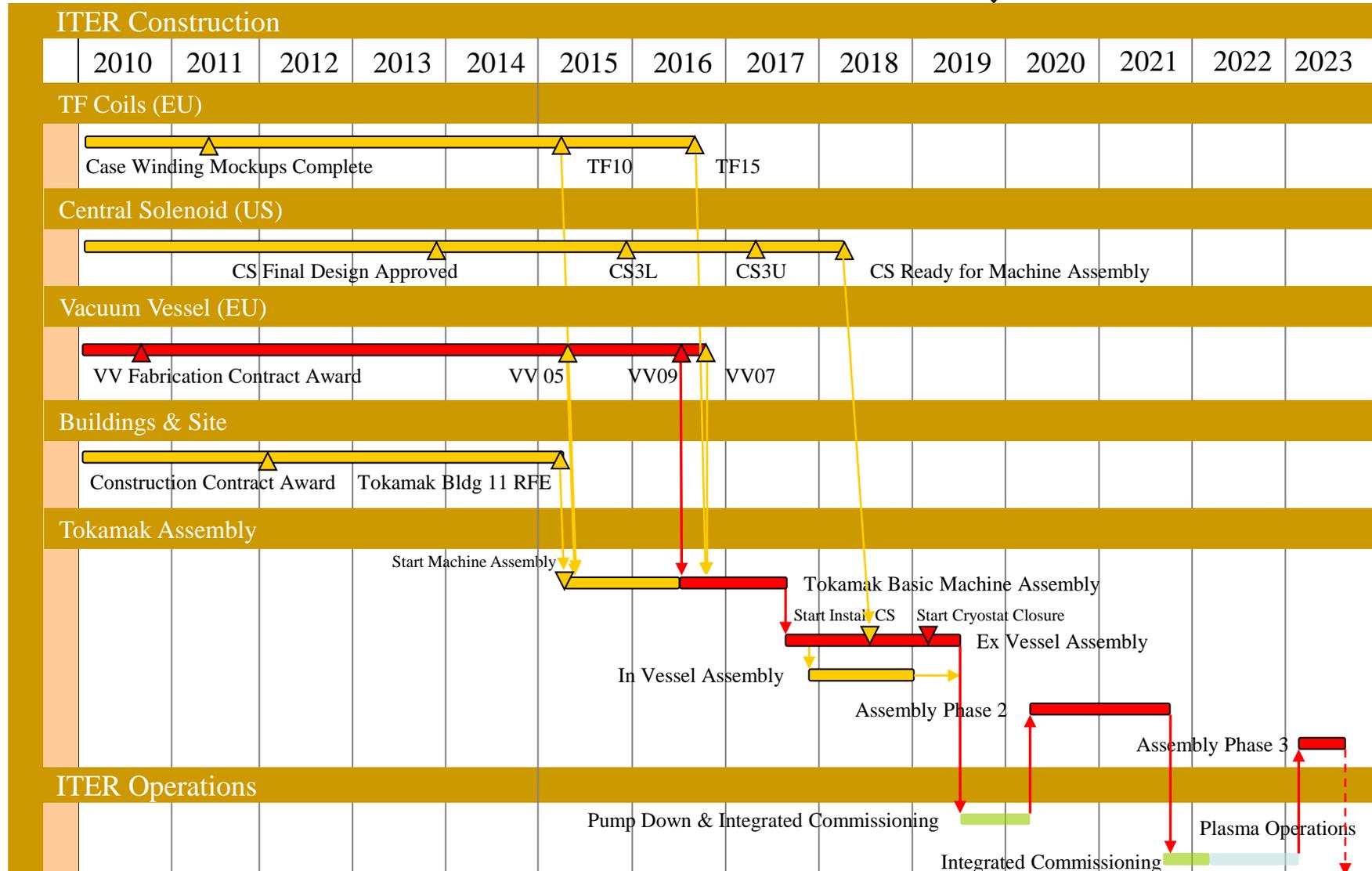


- | | | |
|---|---|---|
| 1. Tokamak building | 14. Main alternating current distribution building | 25. E.T.E. switchyard area |
| 2. Tokamak complex excavation support structure | 15. NIS high voltage power supply building | 26. F.I.E.E. control building |
| 3. Assembly building | 16. Reactive power control building | 27. Cryogenic compressor building |
| 4. Main air building | 17. Reactive power compensator area | 28. Cryogenic receiver building |
| 5. IT building building | 18. Steady state power high voltage substation area | 29. Cryogenic infrastructure |
| 6. Cleaning facility building | 19. Fuel storage tanks (EPS train A) | 30. IT coil fabrication building |
| 7. Hot cell building | 20. Fuel storage tanks (EPS train B) | 31. Site services building |
| 8. Radioactive waste building | 21. Emergency power supply building (train A) | 32. Hot basin and cooling towers |
| 9. Personnel access control building | 22. Emergency power supply building (train B) | 33. Cooling water pumping station |
| 10. Magnet power conversion building 1 | 23. Medium voltage distribution building LC/TA | 34. Heat exchangers |
| 11. Magnet power conversion building 2 | 24. Medium voltage distribution building LC/BB | 35. Control building |
| 12. NB power supply building | | 36. Office building |
| 13. Pulse power high voltage substation area | | 37. Diagnostic building |
| | | 38. IT and switching network control building |
| | | 39. Temporary ITER HQ |



Overview of Construction Schedule

First Plasma



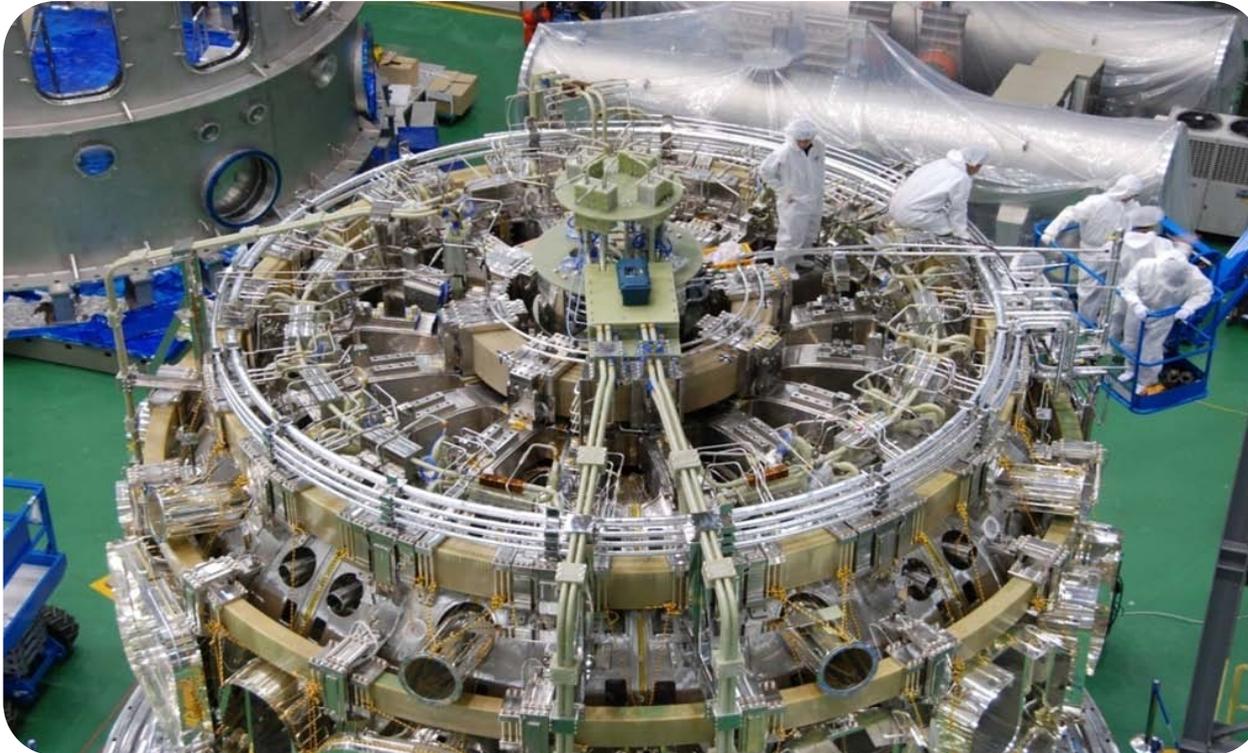


Participation of KSTAR Industrial Partners in ITER Procurement





Challenges and Risk Management thru KSTAR



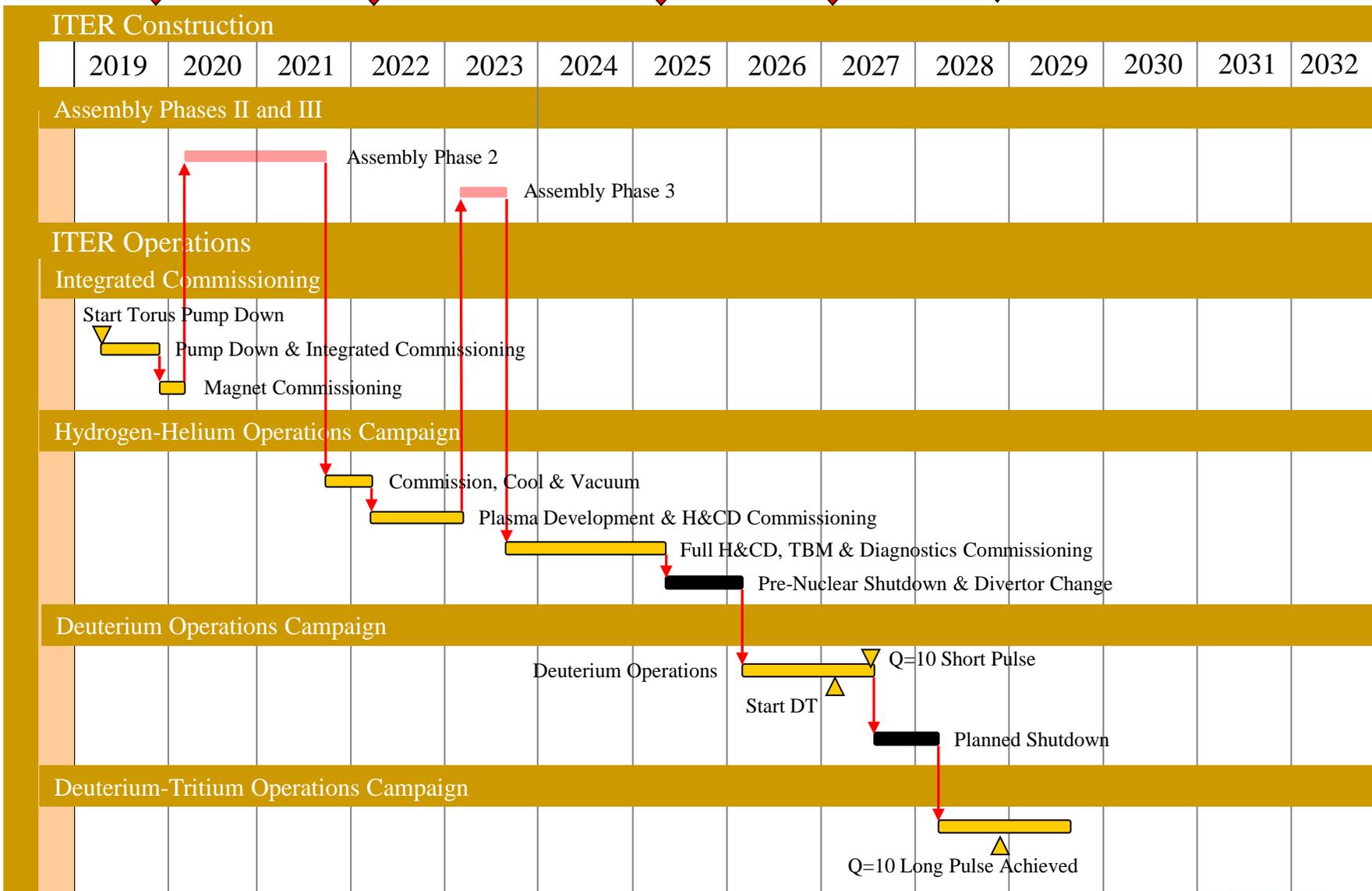
- In-cryostat welding of He lines > 8,500+ points
- In-situ joints of SC Magnets
- Composite material based **Insulation Breakers** to maintain
- He System Operation in AC System

- SC Installation Accuracy for reducing Magnetic Field Error
 - CS Pre-load Assembly for anti-parallel Current Operation
 - Thermal Shield Assembly for maximal Maintainability
- >>> Not by TEST, but by Quality Assurance! Engineering!**



Overview of Operation Schedule

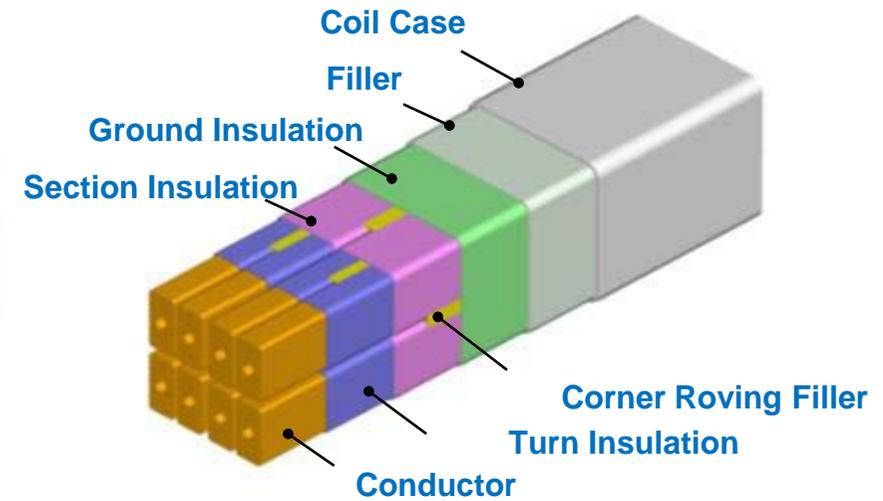
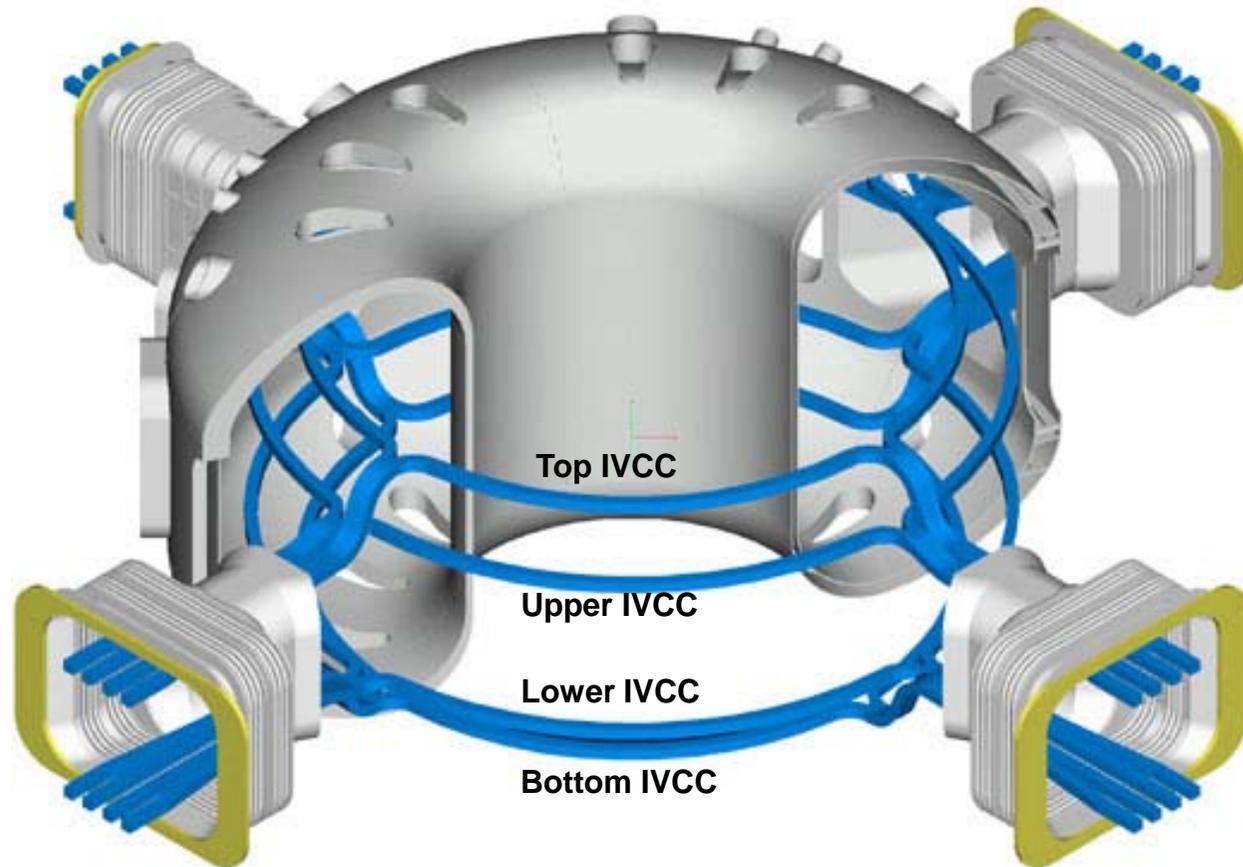
First Plasma Nominal Plasma Hydrogen-Helium Complete Start DT Q=10 Long Pulse Achieved





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In-Vessel Control Coil System (IVCC)



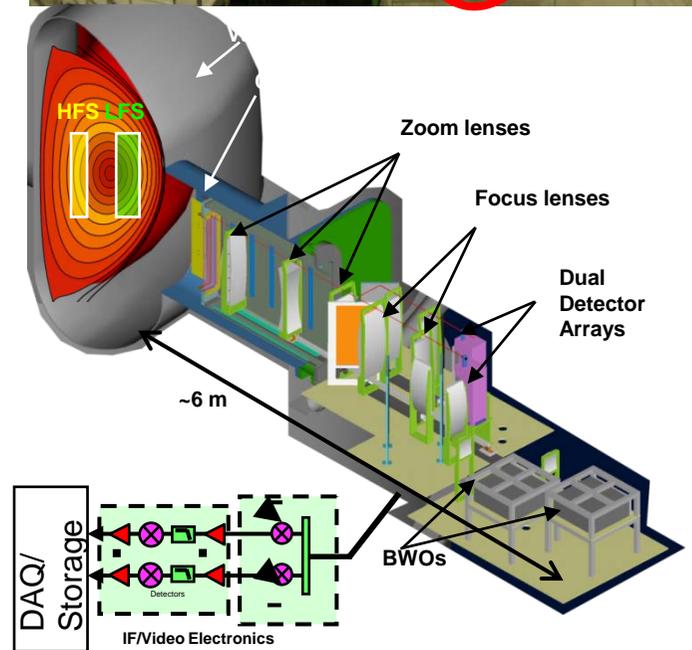
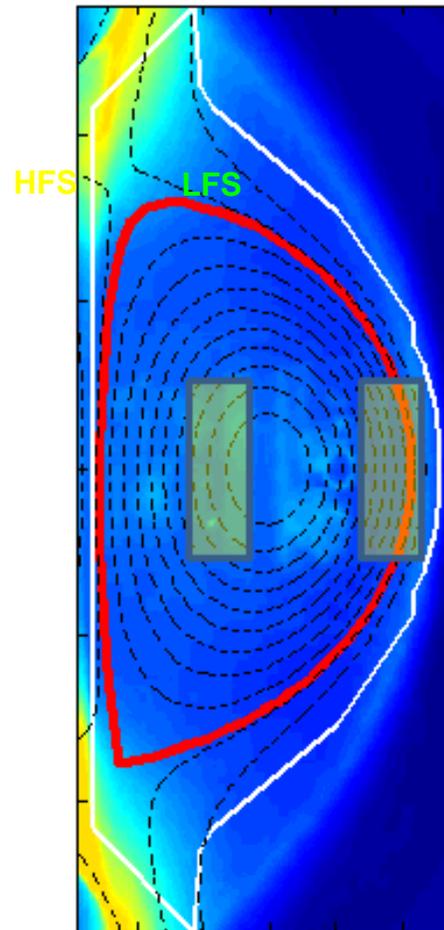
* IVCC : Vertical & Radial Position Control, ELM & RWM Control



KSTAR ECE Imaging (ECEI) System

- ❑ Visualization of physics via 2-D real time imaging
 - ❑ Simultaneous imaging of the Edge and Core T_e fluctuations in KSTAR
 - ❑ Flexible zooming capability for high resolution and global image
- ❑ System Capability
 - ❑ Time resolution ~2 ms
 - ❑ Spatial resolution ~ 1 cm x 1 cm
 - ❑ Signal resolution ~ $dT/\langle T \rangle \sim 2\%$
 - ❑ Pixels ~ 400 channels

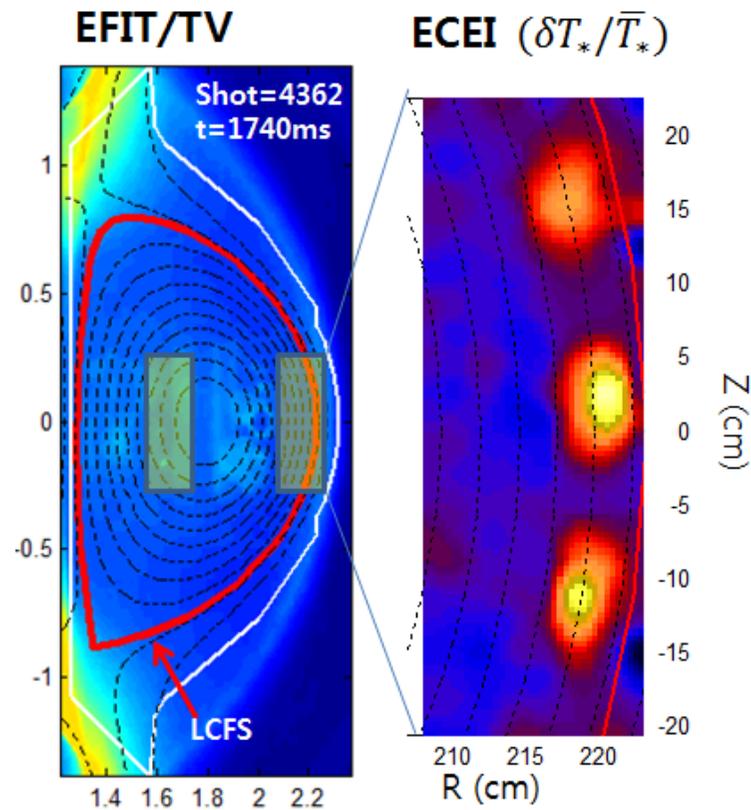
Plasma Coverage



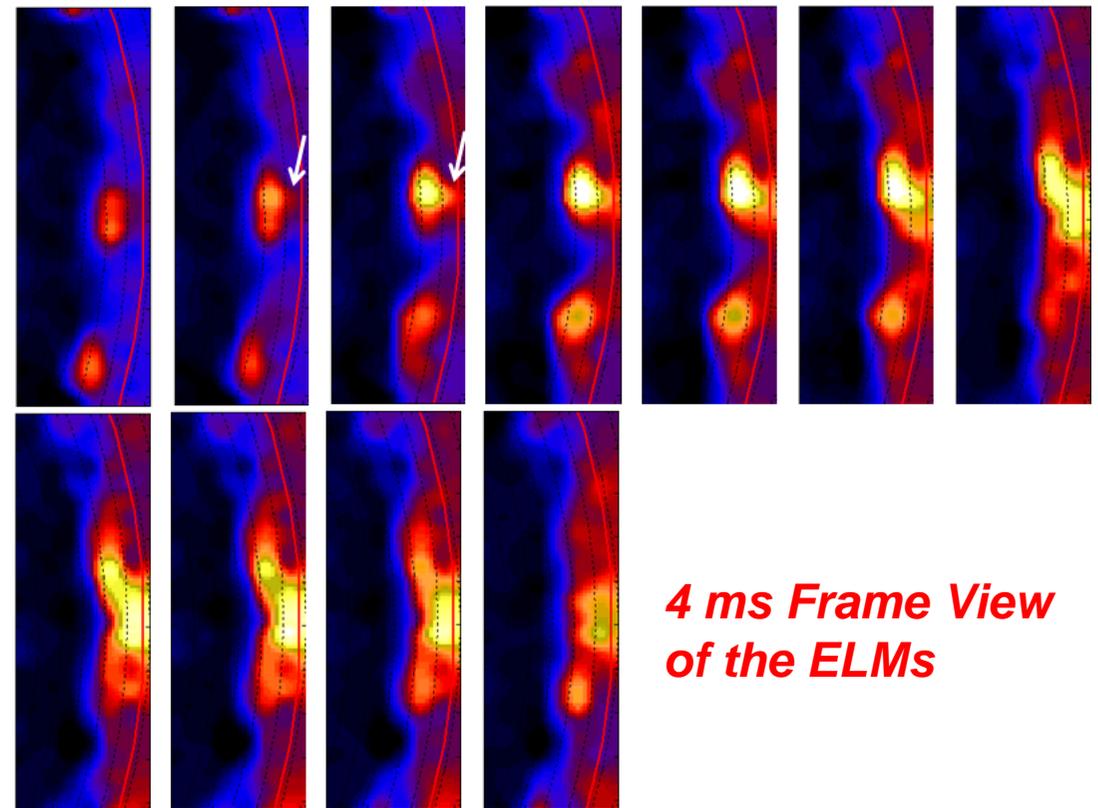


KSTAR ELM Study by ECE Imaging

Filamentary nature of the ELM



Nonlinear crash process of the ELM



*4 ms Frame View
of the ELMs*



KSTAR 5 GHz Klystron for LHCD (ITER Test-bed)

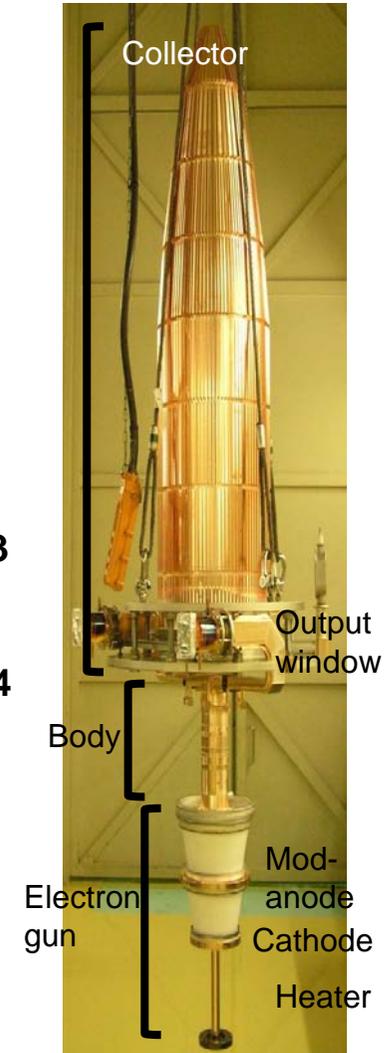


▪ Dummy Loads
(250kW CW each)

▪ Water cooling
manifold

▪ Ion pumps (2 ea.)
▪ Arc detector (4-ch)

- Toshiba prototype Klystron (E3762 RD0)
- Output RF power (Toshiba factory test results)
 - 510 kW, 0.5 s
 - 460 kW, 10 s
 - 300 kW, 10 min.
- Single beam 68 kV, 15 A
- Modulated anode (triode gun)
- Total 6 cavities: 3 input & 3 output cavities 48 dB gain
- 250 kW double windows (BeO) output on VSWR<1.4 any phase
- Collector dissipation: 800 kW
- Body loss < 5 kW
- Window loss < 3.2 kW

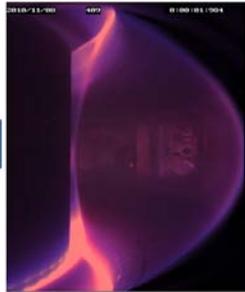
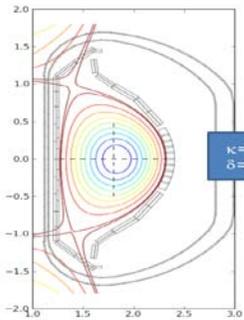


- Test at KSTAR: 20 s @460 kW, 800 s @300 kW
- Utilization for test-bed for ITER waveguide components development



ITER Operation Prep through KSTAR Collaboration

KO-US Collaboration in KSTAR



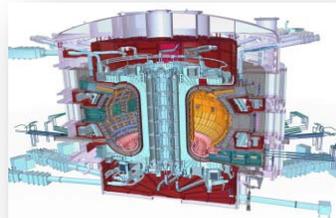
- ▶ **High Performance SS Plasma Operation**
- ▶ **Advanced Scenario** for ITER and beyond
- ▶ **Remote Collaboration** Prep for ITER

▶ Enhance Operational Readiness for ITER by **Joint Operation**

▶ Savings on Substantial Capital Investment for **DEMO R&D**

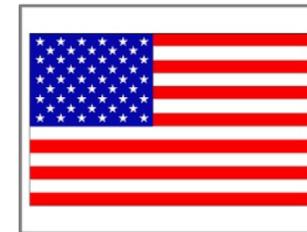
ITER TBM Collaboration

- ▶ **Solid TBM R&D**
- ▶ **DCLL/DFLL TBM**



Synergetic DEMO Consortium

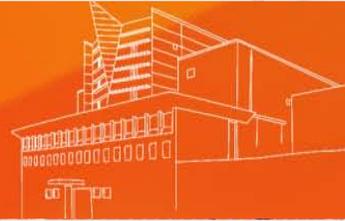
- ▶ **Open Innovation** for DEMO R&D





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New ITER Korea Advanced Research Building





Plasma Application Research Facility



part

3



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Future : DEMO Design and R&D



Fusion Energy Development Promotion Law (FEDPL)

- To establish a **long-term and sustainable legal** framework for fusion energy development phases.
- To **promote industries and institutes** which participating the fusion energy development by supports and benefit.
- The **first country in the world** prepared a legal foundation in fusion energy development.

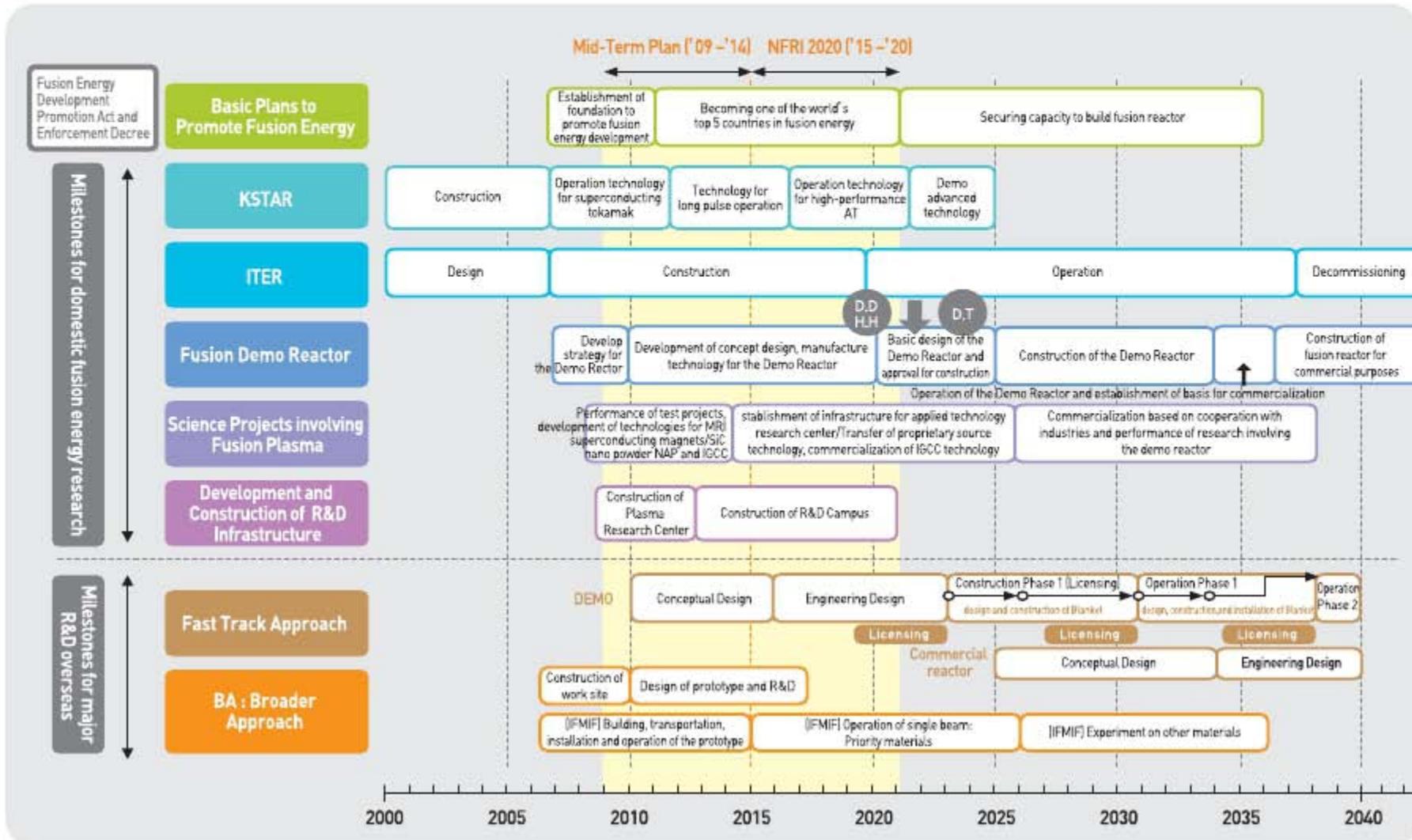
History of the FEDPL

- 1995. 12 : National Fusion R&D Master Plan
- 2005. 12 : National Fusion Energy Development Master Plan
- 2007. 3 : **Fusion Energy Development Promotion Law**
- 2007. 4 : Ratification of ITER Implementation Agreement
- 2007. 8 : Framework Plan of Fusion Energy Development (The First 5-Year Plan)





Strategy for Mid to Long-term R&D for Commercialization of Fusion Power





Plans for Headquarters Campus

Overview

- ▶ Development Duration: 2012-2026 (Construction in three Phases)

3
phases

• Phase 1 (12 - 16)

➡ Construction of the Headquarters

• Phase 2 (17 - 21)

➡ R&D on Fusion Technology

• Phase 3 (22 - 26)

➡ Development of Fusion Reactor

- ▶ Planned Site: International Science Business Belt

Planned Site and Budget

- ▶ Site : 330,000m² (with total floor area of 86,800m²)
- ▶ Project Budget : about 489 Billion KRW
 - HQ Building, Test Facility Building, Basic Research Building, Development Research Building, Mega Science Infrastructures, etc.



Campus Plan for Headquarters





Phased Development Program for K-DEMO

Preparatory Sub-Program
(07~ 11)

DEMO R&D Sub-Program
(12~ 21)

DEMO Design &
Construction Sub-Program
(22~ 36)

Commercial FPP
Construction Sub-Program
(37~ 50)

Strategic Plan of the Program

- Program Definition / Gap Study
- Environmental Analysis for Licensing
- SWOT Analysis for Open Innovation
- Key Strategies & Strategic Initiatives
- Portfolio Management/Cross-Cutting Ideas



Korean Demonstration Fusion Reactor (K-DEMO) aimed for another Success following KSTAR and ITER

Perfect Location for DEMO

- Heavy water reactor producing a large supply of tritium
- Low to intermediate-level radioactive waste repository site nearby
- Equipped with large-capacity power transmission facilities for test





Conceptual Layout Study of K-DEMO Plant





R&D Plan toward DEMO (Proposal)

Fusion DEMO R&D

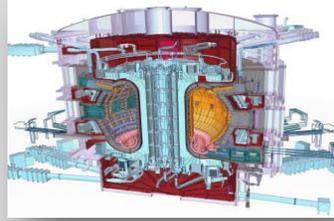


GAP Study

Present Fusion R&D



<KSTAR>



<ITER>



DEMO R&D
Plan

*Open
Innovation*

KO-US, ...
Collaboration
and Post-ITER
Coordination

Broader
Approach
JA-EU

Other
collaboration
scheme

part

4



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Epilogue



Start of Korean Nuclear Power Program



Dr. Syngman Rhee, the founding president of Korea, at the groundbreaking ceremony of the first nuclear research reactor in 1959. (TRIGA)





Exportation of Korean Nuclear Power Plant



- ▶ KEPCO UAE Task Force Team is rejoicing for winning Nuclear Power Plant contract on constructing 4 APR-1400 units in Abu Dhabi, UAE (December 2009)