

Boundary Plasma Interfaces

Working Group Summary, Section 6: Interim Report of the Panel on Program Priorities for the Fusion Energy Sciences Advisory Committee

Boundary Plasma Interfaces Working Group: Chair: S.L. Allen (LLNL) Vice-Chair: M. Ulrickson (SNL)

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Plasma Boundary Interfaces has one key topic: T9



The BPI working group depended strongly on community input



- M. Ulrickson presented a poster at the International Plasma Surface Interactions meeting in May (Portland, Maine)
 - Stimulated many informal discussions
 - (Held every 2 years, a good opportunity)
- A web-based discussion forum was used to gather input from the community:

http://lists.psfc.mit.edu/mailman/listinfo/plasma_boundary

- Very active after the PSI meeting
- Thrusts picked and discussion leaders "volunteered"
- Several calls for input from Priority Panel members:
 - D. Hill, LLNL; A. Hubbard, MIT; M. Ulrickson, SNL

Research approach: Four thrusts corresponding to plasma regions

T9: How do we interface a 100 million degree burning plasma to its room temperature surroundings?



Thrust A discussion leader(referee): A. Hubbard, MIT (M. Fenstermacher, LLNL)

T9-A: Physics of the formation, structure, and stability of the edge transport barrier (robust core coexist with SOL)

- The boundary condition for the core plasma requires high temperature, could mean large ELMs
- The edge transport barrier has strong gradients in temperature and density



- Physics of this region uncertain, represents largest uncertainty in predicting performance of burning plasmas (progress in sheared flow suppression of turbulent transport
- Periodic bursts of heat and particles due to pressure-driven MHD: Edge Localized Modes (ELMs)
- Requires development of integrated models and 2-D measurements
 - High spatial resolution because of strong gradients
 - High temporal resolution required for ELMs

Thrust B discussion leader(referee): B. Labombard, MIT (T. Leonard, GA)

T9-B: Plasma and impurity transport in the scrape-off-layer

- Open field line region outside the plasma core, connected to the material walls
- In a tokamak, op
 Major accomplise
 Plasma en
 Periodic bursts of transported t
 SOL Parallel Transport
 SOL Parallel Cold Plasma "detached"
 Impurity transported to understanding

- In a tokamak, open field lines end on a specially-armored area called the "divertor"
- Major accomplishment of experiments and modeling is "detached divertor"
 Plasma energy is radiated in a low temperature recombining divertor plasma
- Periodic bursts of heat & particles due to Edge Localized Modes (ELMs) are transported to divertor, "Bursty" transport or "Blobs" discovered, models started
- Impurity transport in SOL (and shielding) important part of "radiative divertor"
- Comparison of new measurements with computational models allows understanding the self-consistent relationship between turbulence and transport
- New diagnostics needed to measure ion temperature, plasma flows, and neutral densities flow patterns need to be compared with codes

Divertor Plate

Thrust C discussion leader(referee): P. Stangeby, U. Toronto (D. Whyte, U. Wis.)

T9-C: Tritium retention and plasma material interactions



- Plasma materials interactions include:
 - Collisions of ions with the wall (sputtering) causing erosion
 - Chemical processes (chemical sputtering) causing erosion
 - Deposition of impurities and particles in the wall
 - Erosion takes place at divertor and main chamber wall, impurities enters SOL and can influence core plasma performance
- Tritium can be retained in the walls, particularly with carbon
 - Important for in-vessel inventory, a safety and operations issue
- Understand & control large plasma heat and particle loads -- divertor and main walls
 - Steady-state loads
 - Pulsed loads from ELMs difficult to predict in burning plasma experiment
- New diagnostics needed for flows, heat, and particle flux profiles, and impurity generation also need experiments and modeling of tritium (carbon) transport
- Improved theory and modeling to integrate PMI and SOL modeling

Thrust D discussion leader(referee): M. Ulrickson, SNL (D. Stotler, PPPL)

T9-C: Plasma facing materials and components

- Low-Z solid wall materials (C, Be)
 - Low radiation if leak into core
 - Large database developed on tokamaks and other devices high particle and heat loads have been handled
 - Database developed of fundamental properties
 - Reliable engineering solutions have been found for steady-state high heat flux (absence of neutrons)
 - Tritium retention (in re-deposited material) problem must be addressed
- Medium and High-Z solid wall materials (Molybdenum, Tungsten)
 - Used successfully on several machines
 - Database developed of fundamental properties
 - Some concern on off-normal events (ELMs and disruptions)
 - Reliable engineering solutions have been found for steady-state high heat flux (absence of neutrons)
- Liquid walls
 - Developing database, earlier stage of development
- Both laboratory studies and machine studies required.

Plasma Boundary Interfaces are important for overarching themes

• O2: Burning plasma research

Plasma experiments, modeling and theory have shown:

- Plasma near edge sets boundary conditions which strongly influence transport in the hot core
- Greatest uncertainy in predicting performance of a burning plasma experiment
- Edge Localized Modes (ELMs) important part of power and particle control
- O3: Making fusion power practical
 - Plasma boundary important in overall performance: greatest uncertainty in predictions (now)
 - Combination of SOL plasma and materials must handle heat and particle loads, including pulsed and off-normal events
 - Material choices can have safety, operations, and performance consequences
- O1: Scientific Understanding
 - Complex interaction of turbulence, MHD limits, plasma-surface effects
 - Detailed data required to develop adequate physics models of this region